



Deliverable 10.1: UMAN - Training materials

Work Package 10

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Executive Summary

This report contains the aims, the learning outcomes and the description of the training course organised by the Work Package UMAN on February 14-16 2023 at Bel V (Brussels, Belgium). The report also contains abstracts of the lectures, the slides prepared and presented by each lecture, a career summary of these lecturers and the test organised at the end of the training course.

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1. Introduction

UMAN is a Strategic Study Work Package dedicated to the management of uncertainties potentially relevant to the safety of different radioactive waste management stages and programmes. It includes various activities such as exchanges on views, practices and uncertainty management options and the review of existing strategies, approaches and tools. Interactions between different types of actors including civil society are central to this WP. These interactions are aimed at meeting the shared objective of fostering a mutual understanding of uncertainty management strategies, approaches and preferences. A particular focus is put on uncertainties in direct link with RD&D WPs and with a high (and where relevant medium) priority subdomain of the SRA for which exchanges of information and experiences and strategic studies have been identified as beneficial by the JP actors themselves. The WP considers past and present EU projects on the topics of interest and other initiatives carried out at international level such as IAEA and NEA. The WP allows identifying the contribution of past and on-going RD&D projects to the overall management of uncertainties as well as remaining and emerging issues associated with uncertainty management that could be addressed in subsequent waves or implementation phases of EURAD.

Owing to the nature of a strategic study, it is not foreseen to increase in EURAD the scientific knowledge. Nonetheless, the work carried out in UMAN leads to the development or improvement of strategies, approaches and tools. The knowledge consolidated in UMAN or generated through knowledge/knowledge sharing and discussions in UMAN of common challenging issues constitutes a valuable input to knowledge management activities. Therefore, in collaboration with WP13 (Knowledge Management) of EURAD, UMAN has prepared and organised on 14-16 February 2023 (at Bel V, Brussels, Belgium) a training about Uncertainty Management and some important aspects addressed in UMAN. The present deliverable documents the preparation and the organisation of the course.

1.1 Training aims

The main aim of the training is to address the training need “7.3.1 Treatment of uncertainty” identified by EURAD Work Package 13 as one of the five most urgent and highest priority topics¹. The training will also address other urgent and high priority topics such as “7.1 Safety strategy”, “3.1 Confirm wastefrom compositions, properties and behaviour under storage and disposal conditions, including impact on the disposal environment (wastefrom)” or “3.1.1 Spent Nuclear Fuel”.

1.2 Learning outcomes.

Upon successful completion of this training course, participants should be able to:

- Understand and classify the different types of uncertainties that may need to be managed in a RW disposal programme;
- Explain the links between uncertainty management, the safety case and the decision-making process;
- Use the global UMAN scheme of uncertainty management strategies;
- Explain the main strategies and approaches available to manage uncertainties;
- List the approaches available to perform uncertainty and sensitivity analyses and discuss their pros and cons;
- Grasp the views of Civil Society representatives involved in EURAD on uncertainty management;

¹ See EURAD Deliverable 13.10: *Belmans N., Coeck M., (2020): Title. List of training needs from Research, Development and Demonstration and Strategic Studies Work Packages Final version as of 12.04.2021 of deliverable D13.1 of the HORIZON 2020 project EURAD. EC Grant agreement no: 847593.*

- Understand potentially significant uncertainties related to the waste inventory (with a special focus on problematic wastes, organic-bearing wastes and the radiological characteristics of spent nuclear fuel) and discuss their significance;
- Understand potentially significant uncertainties related to human aspects and discuss their significance;
- Describe and discuss the options available to manage specific examples of uncertainties related to the waste inventory and human-related aspects.

1.3 Training course description

The course is a classroom-based training. The target audience and pre-requisites are the following:

- Participants are required to have a MSc degree;
- The target audience is primarily multidisciplinary (professional) experts (i.e. “generalist experts”). However, the course could also be interesting to more specialized experts;
- Professionals actively involved in waste management programmes;
- Experts involved in the development of safety cases;
- Safety case reviewers (TSOs & regulators).

The schedule of the course organised on 14-16 February 2023 is provided in the table below.

	9:00	10:00	11:00	12:00	13:00	14:00	15:00	16:00	17:00
A Tuesday 14/02/23	Day 1: Uncertainty management strategies, methods & tools								
	A.1 Introduction: Uncertainty management, the safety case & the decision-making process A.2 Types of uncertainties <i>Valéry Dettleux, Bel V</i>		A.3 Uncertainty management strategy and approaches <i>Tim Hicks, GSL</i>	Lunch		A.4 Uncertainty identification and quantification <i>Vincenz Brendler, HZDR</i>		A.5 Approaches to uncertainty and sensitivity analysis – Overview <i>Klaus Röhlig, TUC</i>	A6. Approaches to uncertainty and sensitivity analysis – Practical examples from the DONUT project Dirk Alexander Becker, GRS
B Wednesday 15/02/23	Day 2: Management of uncertainties related to the waste inventory								
	B.1 Characterization and significance of uncertainties related to the waste inventory <i>Attila Baksay, TSENERCON</i>		B.2 Uncertainties in the radiological characterization of spent fuel <i>Peter Schillebeeckx, JRC</i>	Lunch		B.3. Uncertainties associated with waste management routes <i>Chris de Bock, ONDRAF/NIRAS</i>		B.4. Options for the management of uncertainties related to the waste inventory <i>Jeroen Mertens, Bel V</i>	
C Thursday 16/02/23	Day 3: Views of Civil Society & management of uncertainties related to human aspects								
	C.1 Views of Civil Society on Uncertainty Management <i>Julien Dewoghélaëre, NTW</i>		C.2 Characterization and significance of uncertainties related to human aspects <i>Jeroen Bartol, COVRA</i>		C.3 Options for the management of uncertainties related to human aspects (Part 1) <i>Jitka Miksova, SURO</i>	Lunch		C.3 Options for the management of uncertainties related to human aspects (Part 2) <i>Jitka Miksova, SURO</i>	Test

2. Lecture materials

For each lecture of the course this section provides a short synopsis written by the corresponding lecturer. The slides used as support during the lectures are provided in the following 12 appendices.

- Appendix A. Slides of lectures A.1 & A.2
- Appendix B. Slides of lecture A.3
- Appendix C. Slides of lecture A.4
- Appendix D. Slides of lecture A.5
- Appendix E. Slides of lecture A.6
- Appendix F. Slides of lecture B.1
- Appendix G. Slides of lecture B.2
- Appendix H. Slides of lecture B.3
- Appendix I. Slides of lecture B.4
- Appendix J. Slides of lecture C.1
- Appendix K. Slides of lecture C.2
- Appendix L. Slides of lecture C.3

Finally, a short career summary of the lecturers is provided in appendix M and the test organised to verify if the audience has broadly understood the lessons (this was not a “qualification test”) is given in appendix N.

- *Day 1 – Uncertainty management strategies, methods & tools*

Slot of time	9:00 – 9:30
Lecture	A1. Introduction: Uncertainty management, the safety case & the decision-making process A2. Types of uncertainties
Name	Valéry Detilleux
Affiliation	Bel V
Synopsis	Decisions associated with radioactive waste management programmes are made in the presence of irreducible and reducible uncertainties. In the early phase of a programme, several choices must be made on the basis of limited information and need to be confirmed before or during the construction and operation of the disposal facility. At the end of the process, some uncertainties will inevitably remain and it should be demonstrated that these uncertainties do not undermine safety arguments. Hence, the management of uncertainties is a key issue when developing and reviewing the safety case of waste management facilities and, in particular, of waste disposal facilities due to the long-time scales during which the radiotoxicity of the waste remains significant. This introductory lecture will provide a general framework about uncertainty management based on the work performed within UMAN: key definitions, types of uncertainties, links between the management of uncertainties and the decision making process as well as the safety case.

Transition to next talk	This lecture is an introduction to all next lectures and provide an overview of the whole course structure.
Keywords	Uncertainty, risk, Types of uncertainties, Decision making process, Safety Case, Uncertainty management strategy

Slot of time	10:30 – 12:00
Lecture	A3. Uncertainty management strategy and approaches
Name	Tim Hicks
Affiliation	GSL
Synopsis	We need to make choices and decisions through all phases of radioactive waste disposal facility development. There are many uncertainties that we need to be aware of when we make these decisions so that our judgments are as well informed as they can be and we can achieve our desired outcomes. These uncertainties range from those associated with the overall disposal programme and inventory to those associated with the data and models that underpin our safety assessments. By implementing a strategy for managing these uncertainties throughout the lifetime of a disposal facility, we have the best chance of arriving at the optimum solution for the long-term safety of disposal. This lecture discusses the components of an uncertainty management strategy.
Transition to next talk	Any strategy for managing uncertainties will of course include activities to identify, characterise and analyse those uncertainties. The identification and assessment of the safety significance of uncertainties is addressed in A4, A5 and A6.
Keywords	Uncertainty management, radioactive waste disposal, types of uncertainty, treatment of uncertainty.

Slot of time	13:30 – 14:30
Lecture	A4. Uncertainty identification and quantification
Name	Vinzenz Brendler
Affiliation	HZDR
Synopsis	Before entering into uncertainty processing it is essential to identify such uncertainties (here only numerical ones are considered), to rank them according to their relevance, to categorize them, elucidate internal dependencies and cross-interactions. These steps are supported by B1-B4. Also suitable methodologies to analyse and illustrate complex uncertainty patterns are required. Finally, transfer of only qualitatively described uncertainties into numerical ones must be discussed.
Transition to next talk	Once uncertainties are analysed and categorized, they can be used for uncertainty and sensitivity analyses, which is the link to A5 and A6. Respective management decisions are illustrated in B5
Keywords	Numerical uncertainties: models, methodologies, categorization, characterization, Treatment, evaluation

Slot of time	14:45 – 15:45
Lecture	A5. Approaches to uncertainty and sensitivity analysis – Overview
Name	Klaus-Jürgen Röhlig
Affiliation	Clausthal University of Technology (TUC, Germany)
Synopsis	Uncertainty and sensitivity analyses (UA, SA) as addressed in this lecture serve the purpose of quantifying the uncertainty of model simulation results (“indicators”) as well as of the impact input uncertainties have on these output uncertainties. If the input uncertainties are quantified using probability density functions (pdfs, cf. lecture A4), the toolbox of stochastic methods offers a variety of methods for UA and SA. The lecture will focus on stochastic methods which are well-established, but other approaches are also applied.
Transition to next talk	A number of methods introduced in lecture A5 will, by means of example, further be addressed in A6 which will also shed light on technical details.
Keywords	Uncertainty Analysis, Sensitivity Analysis, Probabilistic Methods, Variance-based Methods

Slot of time	16:00 – 17:00
Lecture	A6. Approaches to uncertainty and sensitivity analysis – Practical examples from the DONUT project
Name	Dirk Alexander Becker
Affiliation	GRS
Synopsis	<p>An overview of practical application of uncertainty and sensitivity analysis in numerical performance assessment (PA) is given. It is demonstrated how one can derive sensitivity statements about PA models by means of probabilistic investigations, what one can learn from such investigations about the model behaviour how one can use such knowledge in view of uncertainty management.</p> <p>The first part of the lesson deals with general practical advantages and disadvantages of different types of UA/SA approaches as presented in the previous talk. Then a few studies are presented that are being performed in the framework of the DONUT WP or elsewhere. Finally, some recommendations for executing effective and meaningful probabilistic investigations are given.</p>
Transition to next talk	
Keywords	Uncertainty analysis, sensitivity analysis, numerical simulation

- *Day 2 – Management of uncertainties related to the waste inventory*

Slot of time	9:00 – 10:15
Lecture	B1. Characterization and significance of uncertainties related to the waste inventory
Name	Attila Baksay
Affiliation	TSENERCON
Synopsis	Steps of radioactive waste management (waste generation, characterization, waste processing, etc.) are introduced with regards to the waste inventory. The sources of uncertainties of waste inventory are discussed. Uncertainties connected to each waste management steps are discussed, focusing on their significance from the safety and optimization point view. Possibilities of characterization of uncertainties are presented based on the results of EURAD UMAN work package.
Transition to next talk	A general introduction to next lectures, in particular to B.3 and B.5.
Keywords	Radioactive waste management, Sources of uncertainties of radioactive waste inventory, Significance of uncertainties in waste inventory

Slot of time	10:45 – 12:15
Lecture	B2. Uncertainties in the radiological characterization of spent fuel
Name	Peter Schillebeeckx
Affiliation	JRC
Synopsis	The basic principles of uncertainty evaluation and propagation including the production of covariance matrices are reviewed. The production of radiological quantities such as the decay power and neutron emission in spent nuclear fuel is discussed. The basics behind the theoretical estimation of these quantities together with a realistic uncertainty evaluation is explained. In addition, non-destructive methods to validate the theoretical calculations are presented.
Transition to next talk	
Keywords	Bias, uncertainty propagation, measurement error, systematic effects, covariance matrix, depletion codes, decay heat, neutron emission, nuclear data, disposal

Slot of time	13:30 – 14:30
Lecture	B3. Uncertainties associated with waste management routes
Name	Chris de Bock

Affiliation	ONDRAF/NIRAS
Synopsis	<p>Challenging wastes are wastes for which it is difficult to define an appropriate management route. ROUTES has identified 11 wastes types that are generally considered to be challenging wastes. Main blocking points for defining a management route are: (1) lack of disposal route, (2) characterization issues, (3) treatment or conditioning issues.</p> <p>Characterization uncertainties are the root cause of the issues around legacy waste. Legacy waste risks being entrapped in a double vicious circle around characterization.</p> <p>The absence of an established disposal route is the root cause of the early or delayed conditioning dilemma.</p> <p>Sharing solutions between countries may have substantial benefits, particularly for SIMS. Sharing solutions makes needed infrastructure available, know-how is exchanged and costs are shared. However, prerequisite is a common safety culture and a level playing field between participating countries.</p>
Transition to next talk	Possible management options for uncertainties addressed in this lecture will be presented in lecture B5.
Keywords	Waste management, Characterization, Disposal routes, Legacy waste, Treatment and conditioning, Shared solutions

Slot of time	14:45 – 16:15
Lecture	B4. Options for the management of uncertainties related to the waste inventory
Name	Jeroen Mertens
Affiliation	Bel V
Synopsis	Present the results of the survey on the importance of the different types of uncertainties related to inventory. Present the uncertainties investigated under the 4.2 UMAN task. Highlight methods to deal with those uncertainties. Give some practical examples of uncertainty management.
Transition to next talk	
Keywords	Uncertainty related to waste inventory, radionuclide inventory, chemical composition, physicochemical conditions in storage or disposal

- *Day 3 (16/02/23) – Views of Civil Society & management of uncertainties related to human aspects*

Slot of time	9:00 – 9:45
Lecture	C1. Views of Civil Society on Uncertainty Management
Name	Julien Dewoghélaëre
Affiliation	NTW
Synopsis	<p>Based on work carried out by UMAN Task 5 and the organisation of pluralistic seminars, the lecture will provide elements on Civil Society views regarding uncertainty management with a focus on human aspects.</p> <p>The lecture will present: Model of civil society involvement in EURAD research and in UMAN project, results on seminars related to CS views on uncertainty management (global picture), results related to uncertainties on human aspects, identified methodologies as a way to enable pluralistic management of uncertainties.</p>
Transition to next talk	This lecture will provide a global framework to grasp the lecture C.2.
Keywords	Pluralistic assessment and methodologies, civil society concerns,

Slot of time	10:00 – 11:15
Lecture	C2. Characterization and significance of uncertainties related to human aspects
Name	Jeroen Bartol
Affiliation	COVRA
Synopsis	<p>In this lecture we will present the methods used, lessons learnt and the final results of UMAN Subtask n°3.4 – Characterization and significance of uncertainties for different categories of actors – Uncertainties related to human aspects. It will focus on the ten main uncertainties associated with the following topics and considered as of high-priority for further investigation i.e.:</p> <ul style="list-style-type: none"> • Process for the identification of a workable set of repository requirements • Continuity of the waste management policy along political changes • Robustness of the presently considered safety requirements with regard to the long term • Public acceptance of the repository at potentially suitable or projected location • Schedule to be considered for implementing the different phases of the disposal programme • Robustness of the safety case vis-à-vis sociotechnical factors • New knowledge • Adequacy of safety-related activities safety provisions • Robustness of safety vis-à-vis possible cyber-attacks or programming errors • Availability of well-educated human resources, and relevant experts in radioactive waste management along the repository lifetime until closure.

	Other uncertainties with a lower priority will also be discussed but in less detail. Lastly, some recommendations for potential future actions to address human related issues within the radioactive waste management programmes that still have the greatest uncertainties will be given.
Transition to next talk	The next talk will provide information about how such uncertainties could be managed.
Keywords	Uncertainties related to human aspects, UMAN Subtask n°3.4

Slot of time	11:45 – 12:30 & 14:00 – 14:45
Lecture	C3. Options for the management of uncertainties related to human aspects
Name	Jitka Mikšová
Affiliation	SURO
Synopsis	The lecture is presenting uncertainties related to human aspects and their possible management based on the findings which has been identified and discussed within UMAN tasks work. Four different uncertainties were chosen as seen of a high importance by main actors in the RWM participating in EURAD, i.e. WMO, TSO and RE, completed with CS view. First part of the lecture will be dedicated to the Programme uncertainties, especially to Public acceptance and Schedule of the disposal programme implementation. The second part of the lecture will discuss uncertainties related to a facility construction and the uncertainty related to “new knowledge” which can occur during a long lasting disposal programme.
Transition to next talk	
Keywords	Human aspects, programme uncertainties, public acceptance, schedule, new knowledge

Appendix A. Slides of lectures A.1 & A.2

A.1 Introduction: Uncertainty management, the safety case & the decision-making process

A.2 Types of uncertainties

Prepared by Valéry Detilleux, Bel V



INTRODUCTION: Uncertainty management, the safety case & the decision-making process (A1) Types of uncertainties (A2)

EURAD Training on Uncertainty Management, A1 & A2

Valéry Detilleux • Bel V



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Introduction: Uncertainty management, the safety case & the decision-making process

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KEY WORDS

- Uncertainty, risk
- Types of uncertainties
- Decision making process
- Safety Case
- Uncertainty management strategy

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Introduction: Uncertainty management, the safety case & the decision-making process

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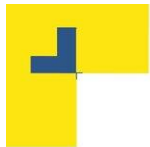


CONTENT

- Introduction
 - What is UMAN ?
 - Objective and agenda of the course
 - Learning outcomes
 - Go around the table
- Uncertainty management, the safety case & the decision-making process
 - Definitions (uncertainty, risk)
 - Uncertainty and the decision making process
 - Uncertainty and the safety case
 - Types and classification of uncertainties
 - Elements of an uncertainty management strategy
- Summary and conclusions

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Introduction: Uncertainty management, the safety case & the decision-making process



INTRODUCTION

- What is UMAN ?
- Objective and agenda of the course
- Learning outcomes
- Roundtable

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Introduction: Uncertainty management, the safety case & the decision-making process





EURAD JOINT PROGRAMME VISION

A step change in European collaboration towards safe radioactive waste management (RWM) through the development of a robust and sustained science, technology and knowledge management programme that supports timely implementation of RWM activities and serves to foster mutual understanding and trust between participants

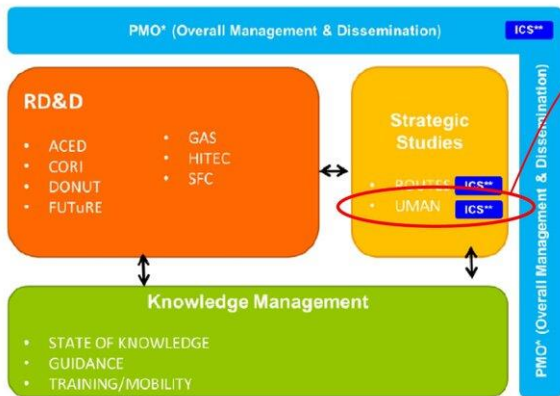
<https://www.ejp-eurad.eu>

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UMAN: AN EURAD ACTIVITY



The **Uncertainty Management multi-Actor Network**

*Programme Management Office
** Interactions with Civil Society

February 14, 2023

Introduction: Uncertainty management, the safety case & the decision-making process





UMAN: RATIONALE

- Decisions associated with RWM programmes are made in the presence of **irreducible and reducible uncertainties**
- Several choices made on the basis of **limited information** in early programme phases may have to be confirmed before or during the construction and operation of the facility
- The Council Directive 2011/70/EURATOM requires that transparency be provided by ensuring effective public information and opportunities for all stakeholders concerned to **participate in the decision-making process**
- At the end of the process, uncertainties will inevitably remain and it should be demonstrated that these **uncertainties do not undermine safety arguments**
- Hence, the **management of uncertainties is a key element** of successful programme planning and of the safety case of waste management facilities and...
- ...in particular, of waste disposal facilities due to the long time scales during which the radiotoxicity of the waste remains significant

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Introduction: Uncertainty management, the safety case & the decision-making process



UMAN: OBJECTIVES

- Develop a **common understanding** among different categories of actors (WMOs, TSOs, REs & Civil Society) **on uncertainty management** and how it relates to risk & safety
- In cases where a common understanding is beyond reach, to achieve mutual understanding on **why views** on uncertainties and their management **are different** for various actors
- **Share knowledge/know-how** and discuss common methodological/strategical **challenging issues**
- Identify the **contribution of past & on-going R&D projects** to the overall management of uncertainties
- Identify **remaining and emerging issues and needs**

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Introduction: Uncertainty management, the safety case & the decision-making process





UMAN: PARTICIPANTS

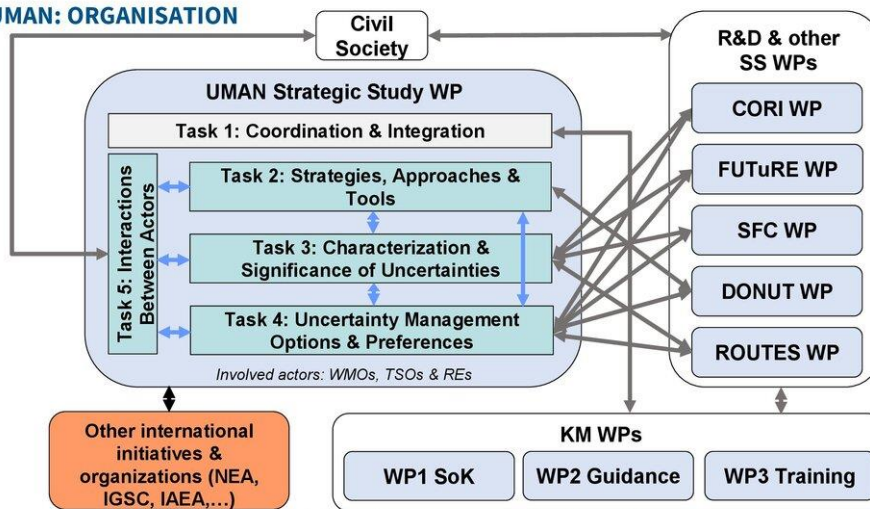
Belgium	ONDRAF, BEL V, SCK•CEN	Netherlands	COVRA, NRG
Bulgaria	INSTITUTUL NATIONAL DE ENERGIE NUCLEARE	Romania	RATEN ICN PITEȘTI
Czech Republic	SÚRAO, SÚRO	Slovakia	STU
Finland	VTT, EnviroCase	Slovenia	Jožef Stefan Institute Ljubljana, Slovenia, ERM
France	ANDRA, IRSN, CIRS, MUZADIS, NTW	Spain	enresa, Ciemat
Germany	BGE, GRS, KIT, TU Clausthal, HEDR	Switzerland	nagra, PAUL SCHERRER INSTITUT, PSI
Hungary	TS ENERCON Engineering and Consulting	UK	Radioactive Waste Management, Galson Sciences Ltd
Lithuania	CENTER for Environmental Research, LEI	Ukraina	STATE SCIENTIFIC AND TECHNOLOGICAL CENTER FOR RADIOACTIVE WASTE MANAGEMENT, SSTC NRS

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UMAN: ORGANISATION



February 14, 2023

Introduction: Uncertainty management, the safety case & the decision-making process





UMAN SUBTASK 1.2: TRAINING

- The main aim of the training is to address the training need “7.3.1 Treatment of uncertainty” identified in EURAD deliverable D13.1 as one the five most urgent and highest priority topics.
- The training will also address other urgent and high priority topics such as:
 - “7.1 Safety strategy”,
 - “3.1 Confirm wastefrom compositions, properties and behaviour under storage and disposal conditions, including impact on the disposal environment (wastefrom)” or
 - “3.1.1 Spent Nuclear Fuel”.

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Introduction: Uncertainty management, the safety case & the decision-making process



PROGRAMME OF THE COURSE

A journey into the world of uncertainties



	9:00	10:00	11:00	12:00	13:00	14:00	15:00	16:00	17:00	
Day 1: Uncertainty management strategies, methods & tools										
A Tuesday 14/02/23	A.1 Introduction: Uncertainty management, the safety case & the decision-making process <i>Valéry Delbecq, Bel V</i>		A.3 Uncertainty management strategy and approaches <i>Tim Hicks, GSI</i>		Lunch	A.4 Uncertainty and identification and quantification <i>Vincent Brendler, HZDR</i>		A.5 Approaches to uncertainty and sensitivity analysis - Overview <i>Klaus Böhlig, TUC</i>		
	A.2 Types of uncertainties		A.6 Approaches to uncertainty and sensitivity analysis - Practical examples from the DONUT project <i>Dirk Alexander Becker, GRS</i>							
Day 2: Management of uncertainties related to the waste inventory										
B Wednesday 15/02/23	B.1 Characterization and significance of uncertainties related to the waste inventory <i>Abla Epsey, TSENERCON</i>		B.2 Uncertainties in the radiological characterization of spent fuel <i>Peter Schillebeeckx, JRC</i>		Lunch	B.3 Uncertainties associated with waste management routes <i>Ching-Ida Boak, ONDRAFANRAS</i>		B.5. Options for the management of uncertainties related to the waste inventory <i>Jeroen Mertens, Bel V</i>		
	C.1 Views of Civil Society on Uncertainty Management <i>Julien Demoghebaire, NTW</i>		C.2 Characterization and significance of uncertainties related to human aspects <i>Jeroen Baelis, COVRA</i>		C.3 Options for the management of uncertainties related to human aspects (Part 1) <i>Jiří Mikšov, SURFO</i>		C.3 Options for the management of uncertainties related to human aspects (Part 2) <i>Jiří Mikšov, SURFO</i>		Test	Feedback
Day 3: Views of Civil Society & management of uncertainties related to human aspects										
C Thursday 16/02/23	C.1 Views of Civil Society on Uncertainty Management <i>Julien Demoghebaire, NTW</i>		C.2 Characterization and significance of uncertainties related to human aspects <i>Jeroen Baelis, COVRA</i>		C.3 Options for the management of uncertainties related to human aspects (Part 1) <i>Jiří Mikšov, SURFO</i>		C.3 Options for the management of uncertainties related to human aspects (Part 2) <i>Jiří Mikšov, SURFO</i>		Test	Feedback
	C.1 Views of Civil Society on Uncertainty Management <i>Julien Demoghebaire, NTW</i>		C.2 Characterization and significance of uncertainties related to human aspects <i>Jeroen Baelis, COVRA</i>		C.3 Options for the management of uncertainties related to human aspects (Part 1) <i>Jiří Mikšov, SURFO</i>		C.3 Options for the management of uncertainties related to human aspects (Part 2) <i>Jiří Mikšov, SURFO</i>		Test	Feedback

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Introduction: Uncertainty management, the safety case & the decision-making process





LEARNING OUTCOMES (FULL COURSE)

Upon successful completion of this training course, participants should be able to:

- Understand and classify the different types of uncertainties that may need to be managed in a RW disposal programme
- Explain the links between uncertainty management, the safety case and the decision-making process
- Use the global UMAN scheme of uncertainty management strategies
- Explain the main strategies and approaches available to manage uncertainties
- List the approaches available to perform uncertainty and sensitivity analyses and discuss their pros and cons
- Grasp the views of Civil Society representatives involved in EURAD on uncertainty management
- Understand potentially significant uncertainties related to the waste inventory (with a special focus on problematic wastes, organic-bearing wastes and the radiological characteristics of spent nuclear fuel) and discuss their significance
- Understand potentially significant uncertainties related to human aspects and discuss their significance
- Describe and discuss the options available to manage specific examples of uncertainties related to the waste inventory and human-related aspects

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LEARNING OUTCOMES (LESSONS A1, A2)

Upon successful completion of this training course, participants should be able to:

- Understand and classify the different types of uncertainties that may need to be managed in a RW disposal programme
- Explain the links between uncertainty management, the safety case and the decision-making process
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- Explain the main strategies and approaches available to manage uncertainties
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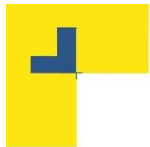


LET'S QUICKLY PRESENT OURSELVES



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Introduction: Uncertainty management, the safety case & the decision-making process



UNCERTAINTY MANAGEMENT, THE SAFETY CASE & THE DECISION-MAKING PROCESS

- Definitions (uncertainty, risk)
- Uncertainty and the decision making process
- Uncertainty and the safety case
- Types and classification of uncertainties
- Elements of an uncertainty management strategy

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Introduction: Uncertainty management, the safety case & the decision-making process





WHAT DOES « UNCERTAINTY » MEAN ?



“A situation in which something is not known, or something that is not known or certain”

Source: <https://dictionary.cambridge.org>

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« UNCERTAINTY » VS. « RISK »

- **Uncertainty ≠ Risk !**
- **Risk**
 - related to a scenario or sequence of events
 - = Probability x Consequences
 - can be interpreted as the measure of significance of an uncertainty
- **The significance of uncertainties needs to be assessed**



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TYPES OF UNCERTAINTIES: « EPISTEMIC » VS. « ALEATORY »

An uncertainty can be:

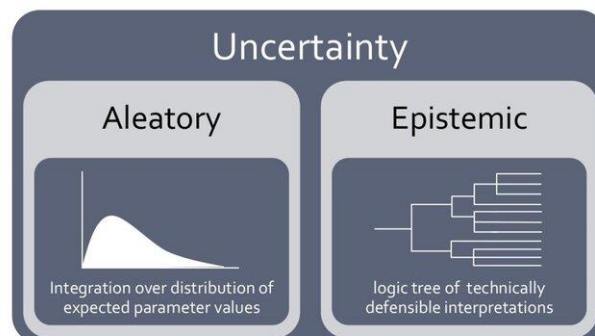
- « **epistemic** »
 - i.e. relating to knowledge or to the degree of its validation
 - e.g. lack of knowledge about site characteristics
 - *can be reduced*
- « **aleatory** »
 - i.e. related to random variability
 - e.g. uncertainty over the time of occurrence or magnitude of rare events
 - *cannot be reduced*

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TYPES OF UNCERTAINTIES: « EPISTEMIC » VS. « ALEATORY »



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UNCERTAINTY & DECISION-MAKING PROCESS

- Disposal programme = “**Stepwise decision-making process**”
- Decisions are made:
 - in presence of **irreducible and reducible uncertainties**
 - considering that some uncertainties will **decrease as new information will become available** e.g. “as-built” properties, monitoring data, R&D results,...
- **Choices** made on the basis of limited information in early phases **may have to be confirmed** during subsequent phases
- **Information about uncertainties and perspectives on how they can be managed** form an important input for the decisions to be taken at each phase

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ROLES OF THE SAFETY CASE

- The safety case provides the **basis for demonstration of safety and for licensing**
- They **assist and guide decisions made at each programme phase**
- The safety case will also be the main basis on which:
 - **dialogue** with interested parties will be conducted
 - **confidence in the safety** of the disposal facility will be developed

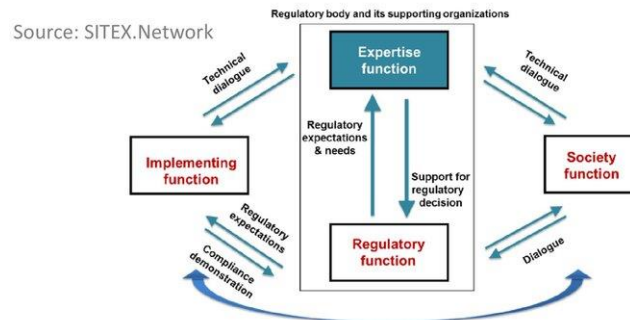
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SAFETY CASE INTERESTED PARTIES



EURAD key actors

WMO: Implementing function

TSO: Expertise function

RE: Could intervene in implementing, society and expertise functions

CS: Society function

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UNCERTAINTY & THE SAFETY CASE

=> The safety case needs to:

- describe the **approach to managing uncertainties**
- acknowledge and **identify significant uncertainties**
- **assess the impact** of uncertainties
- show that **existing levels of uncertainty are acceptable** given the decision(s) to be taken
- show **how significant uncertainties will be managed** in subsequent phases
- be **regularly updated** so that it remains an adequate basis for making decisions



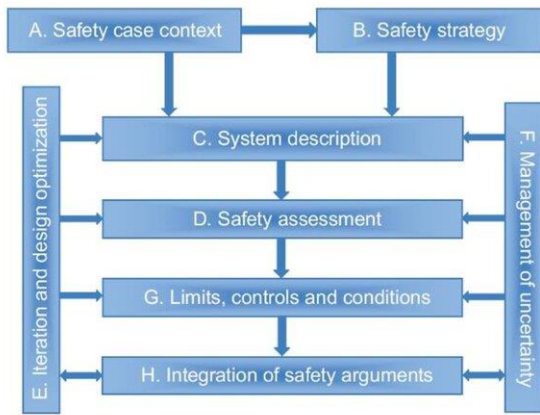
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COMPONENTS OF THE SAFETY CASE



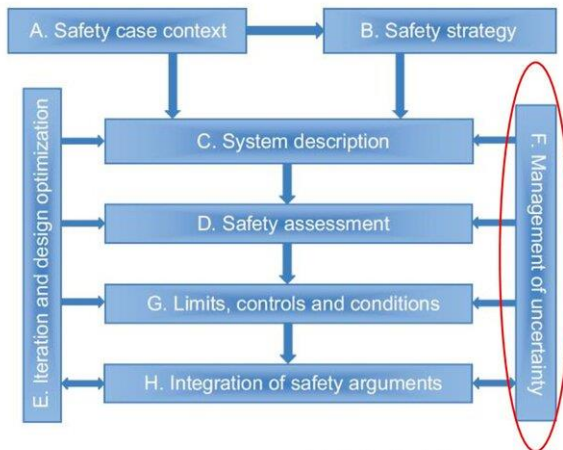
Source: IAEA Safety Guide SSG-23

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MANAGEMENT OF UNCERTAINTY



- Uncertainty management is a **cross-cutting element of the safety case**

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TYPES OF UNCERTAINTIES: UMAN CLASSIFICATION (LEVEL 1)

- Views of a large number of actors representing WMOs, TSOs and REs on the uncertainties associated with radioactive waste disposal safety where collected via UMAN.
- This led to a three-level classification scheme of uncertainties, integrating all points of views and covering all stages of a RWM programme.
- This scheme distinguishes 5 main types of uncertainties:
 - Programme uncertainties**, associated with the RWM programme and other prevailing circumstances (societal, resources, etc.);
 - Uncertainties associated with the **initial characteristics** of the disposal system and its environment;
 - Uncertainties associated with the **evolution of the disposal system and its environment**, which include effects of events and processes that may affect the initial characteristics (e.g. uncertainties associated with the radiotoxic and chemotoxic elements) as well as human influence or intrusion;
 - Uncertainties related to **concepts (models) and parameters (data)** used in the safety assessment;
 - Uncertainties associated with the **completeness of the safety assessment** (uncertainty in overlooking certain aspects relevant to safety).

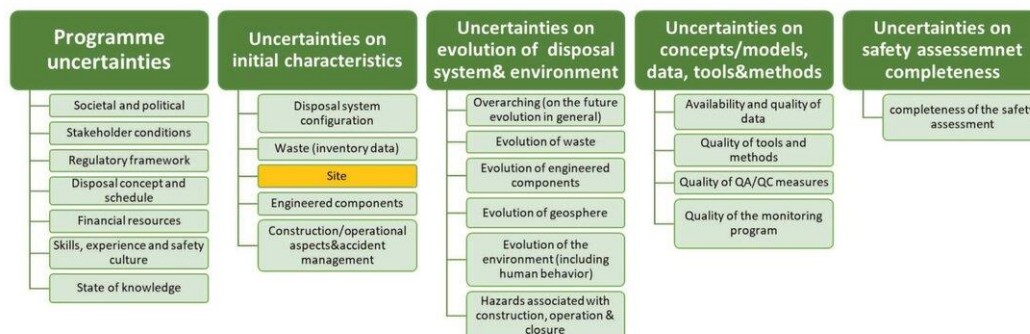
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TYPES OF UNCERTAINTIES: UMAN CLASSIFICATION (LEVEL 2)

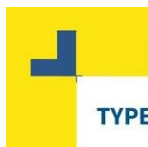
- Each type of uncertainty is grouped in topical groups of uncertainties, which represent the second level of the UMAN classification.



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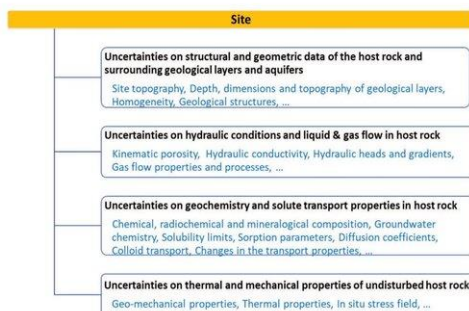
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TYPES OF UNCERTAINTIES: UMAN CLASSIFICATION (LEVEL 3)

- The third level of the classification scheme includes the uncertainties potentially significant for the disposal safety of each level 2 topical group.
- Below an illustration of the third level for the uncertainties associated with the site and geosphere, as identified in UMAN.

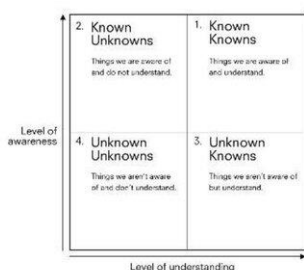


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UNCERTAINTIES & AVAILABILITY OF KNOWLEDGE



From the point of view of the availability of knowledge, uncertainties can belong to one of the three following categories:

- 2: we know what we don't know (known unknowns)
- 3: we don't know that knowledge exists or we ignore existing knowledge (unknown/ignored knowns)
- 4: we don't know we don't know (unknown unknowns).

The last two circumstances represent the uncertainties associated with the completeness of the safety assessment.

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TYPES OF UNCERTAINTIES: UMAN MATRIX

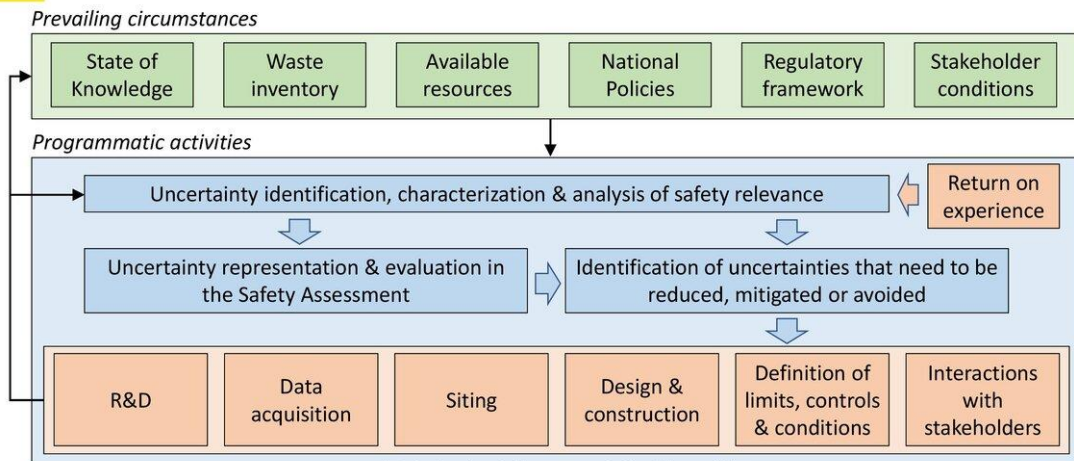
		5. Uncertainties associated with FEP completeness			
Knowledge is available	Lack of knowledge		Known unknowns	Unknown/Ignored Knowns	Unknown Unknowns
Known Knowns <i>What is known & used</i>	Known Unknowns <i>What we know we don't know</i>	1. Programme uncertainties			
		2. Uncertainties associated with initial characteristics			
		3. Uncertainties in the evolution of the disposal system & its environment			
		4. Uncertainties associated with data, tools & methods used in the safety case			
Unknown/Ignored Knowns <i>What is known but we are not aware of or do not consider</i>	Unknown Unknowns <i>What we don't know we don't know</i>				

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ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY



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SUMMARY AND CONCLUSIONS

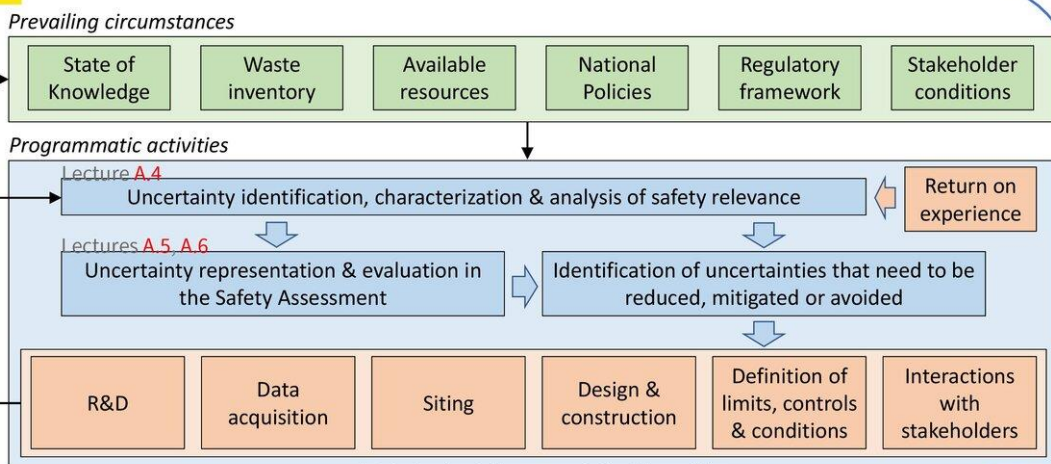
- “Uncertainty is the only certainty there is, and knowing how to live with insecurity is the only security” (John Allen Paulos)
- Uncertainty ≠ Risk
- Managing uncertainties is key when developing a disposal system and assessing its safety
- Various types of uncertainties need to be managed in a disposal programme
- Several options might be available to reduce, avoid or mitigate these uncertainties
- Uncertainty management is an iterative process associated with the stepwise implementation of the disposal programme

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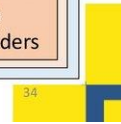


NEXT LECTURES: A JOURNEY INTO THE WORLD OF UNCERTAINTIES (DAY 1)



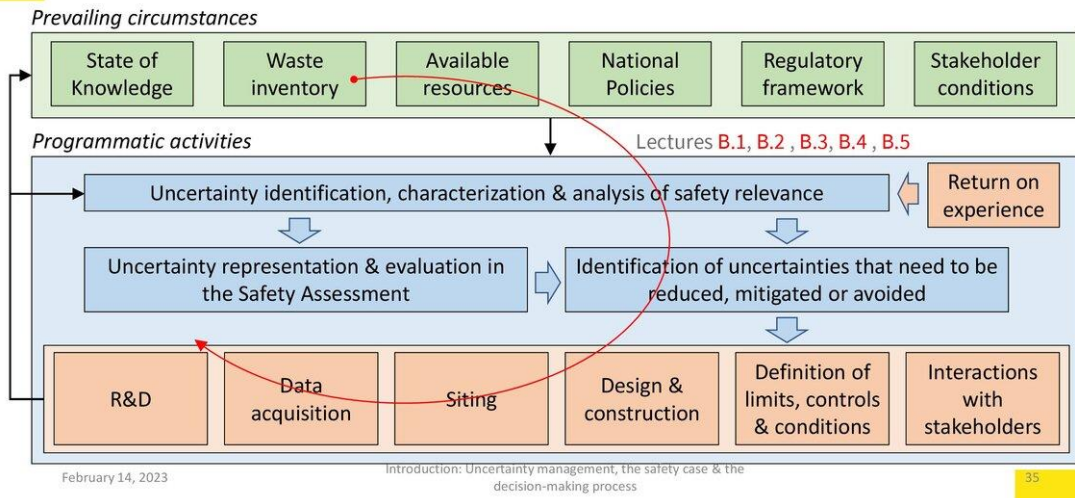
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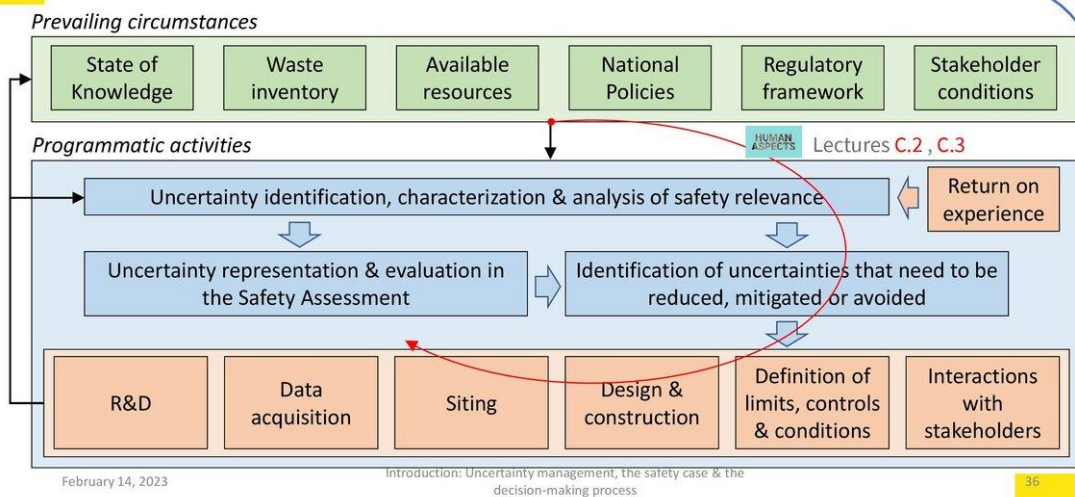




NEXT LECTURES: A JOURNEY INTO THE WORLD OF UNCERTAINTIES (DAY 2)



NEXT LECTURES: A JOURNEY INTO THE WORLD OF UNCERTAINTIES (DAY 3)





BIBLIOGRAPHY AND/OR REFERENCES.

- UMAN - a pluralistic view of uncertainty management, Daniela Diaconu, Valéry Detilleux, Agnieszka Strusinska-Correia and Astrid Göbel, Julien Dewoghelaere, and Dirk-Alexander Becker, EPJ Nuclear Sci. Technol. 9, 2 (2023) <https://doi.org/10.1051/epjn/2022049>

+ References therein.

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Introduction: Uncertainty management, the safety case & the decision-making process



Appendix B. Slides of lecture A.3

A.3 Uncertainty management strategy and approaches

Prepared by Tim Hicks, GSL



UNCERTAINTY MANAGEMENT STRATEGY AND APPROACHES

EURAD Training on Uncertainty Management, A3

Tim Hicks, Galson Sciences Ltd • WP 10



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

Date

Training material

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KEY WORDS.

- Uncertainty management, radioactive waste disposal, types of uncertainty, treatment of uncertainty

Date

Training material



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CONTENTS

- Learning outcomes.
- Introduction.
- Safety case for disposal.
- Elements of an uncertainty management strategy.
- Managing uncertainty throughout a disposal programme.
- Regulatory framework.
- Types of uncertainty.
- Avoidance, mitigation and reduction of uncertainty
 - disposal programme.
 - inventory.
 - disposal concept.
 - as-built conditions.
 - disposal system evolution.
 - data and models.
 - FEPs and scenarios.
- Summary and conclusions.
- Bibliography and/or References.

Date

Training material



LEARNING OUTCOMES.

After the completion of this training unit/lesson, the students/participants should be able to

- set out the components of an uncertainty management strategy
- understand how managing uncertainties is important to decision-making as a disposal programme progresses
- understand the types of uncertainty to be considered in a safety assessment and how they can be reduced, mitigated and avoided

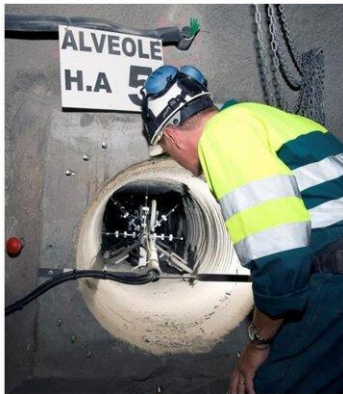
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INTRODUCTION



- We need to make choices and decisions through all phases of radioactive waste disposal facility development.
- There are many uncertainties that we need to be aware of when we make these decisions so that our judgments are as well informed as they can be and we can achieve our desired outcomes.
- Uncertainties range from those associated with the overall disposal programme and inventory to those associated with the data and models that underpin our safety assessments.
- By implementing a strategy for managing these uncertainties throughout the lifetime of a disposal facility, we have the best chance of arriving at the optimum solution for the long-term safety of disposal.
- This lecture discusses the components of an **uncertainty management strategy**.

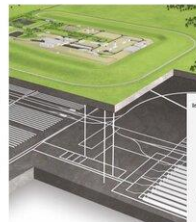
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BACKGROUND

- The IAEA established the fundamental safety objective for all circumstances that give rise to radiation risks:
'The fundamental safety objective to protect people and the environment from harmful effects of ionising radiation'
- We need to dispose of radioactive waste in a way that ensures that this objective is met
- We do this by designing, operating and closing disposal facilities in a way that **isolates and contains** the waste sufficiently to ensure safety
 - operational safety is achieved through engineering features and operational controls
 - post-closure safety is achieved through engineered and natural barriers (safety concept) that contribute to **passive safety** after facility closure
- The safety of disposal is demonstrated in a safety case



"The safety case is the collection of scientific, technical, administrative and managerial arguments and evidence in support of the safety of a disposal facility" IAEA SSG-23

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COMPONENTS OF A SAFETY CASE



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ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY

- An uncertainty management strategy generally involves the following steps
 - identification and characterisation of uncertainties to be managed
 - analysis of the safety relevance of uncertainties
 - representation of safety-relevant uncertainties in the safety assessment
 - identification of uncertainties that need to be reduced, mitigated or avoided based on assessment results
 - actions to avoid, mitigate or reduce uncertainties
- **Views developed as part of UMAN Subtask 2.1, including questionnaire on uncertainty management sent to UMAN participants and literature review**

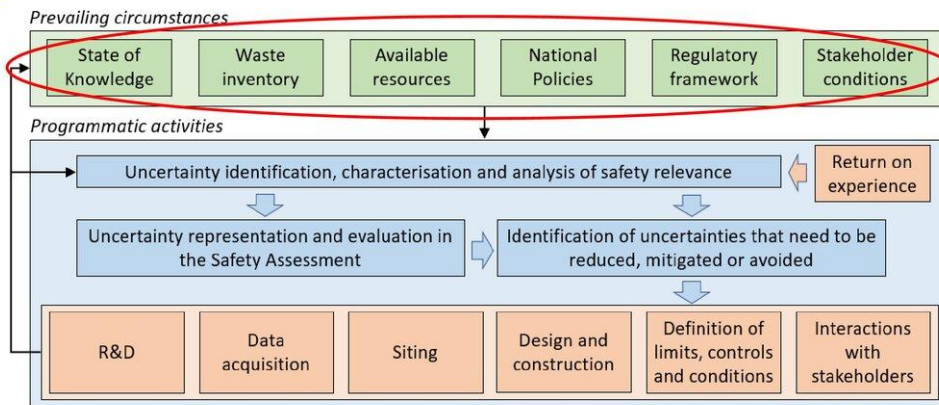
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ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY



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UNCERTAINTY MANAGEMENT AS THE DISPOSAL PROGRAMME PROGRESSES



- Safety cases are produced to support decisions in different programme phases
 - An **initial safety case** is typically produced in the framework of a pre-licensing process in support of:
 - disposal concept development
 - site selection
 - dialogue with stakeholders
 - A **complete safety case** is submitted in the framework of a licence application for the construction and operation of a disposal facility
 - **Safety case updates** are likely to be needed during operation of the facility (e.g. to account for as-built properties, waste characteristics, RD&D, monitoring, design changes) and prior to facility closure
- How uncertainties are accepted and managed evolves throughout these programme phases, with various methods used to **avoid, mitigate or reduce uncertainties**
- The safety case should include a description of the programme for **uncertainty management** consistent with the prevailing circumstances in the disposal programme

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UNCERTAINTY MANAGEMENT AS THE DISPOSAL PROGRAMME PROGRESSES

- An uncertainty management strategy is thus an important component of a radioactive waste disposal programme
 - information about uncertainties and how they can be managed informs decisions to be taken at any programme step
 - how much uncertainty can be accepted at a given step depends on the decisions to be taken
- Each step in the identification, characterisation, assessment and treatment of uncertainties is updated progressively and iteratively as the disposal programme progresses
 - iterative safety assessments to understand parameters to which performance measures are most sensitive, which **can guide uncertainty reduction** through data acquisition activities
 - iterative approach to research and data acquisition activities aimed at **reducing or mitigating uncertainties**

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UNCERTAINTY MANAGEMENT AS THE DISPOSAL PROGRAMME PROGRESSES

- At each major step of a disposal programme, the safety case will need to explain
 - whether any remaining uncertainties are significant for safety; residual uncertainties will remain at any licensing stage, but it is important to show that they will not undermine the goals of the present stage and can be dealt with as necessary in future stages
 - why it is appropriate to move to the next stage of facility development despite these uncertainties
 - the strategies to be employed to address remaining uncertainties
- Each revision of the safety case should therefore include a plan for further work to address significant unresolved issues, in particular to reduce significant remaining uncertainties or to reduce their relevance or avoid them entirely
 - **reducible uncertainties** that have a **substantial influence** on the results of a safety assessment would need to be addressed through, for example, further research, improved modelling, or design modifications
 - **irreducible uncertainties** that have a **substantial influence** on the results of a safety assessment would need to be addressed through, for example, further analysis of specific scenarios, conservative assumptions, or probabilistic assessments
 - **uncertainties that have a minor influence** on the results of a safety assessment may not need further consideration

Date

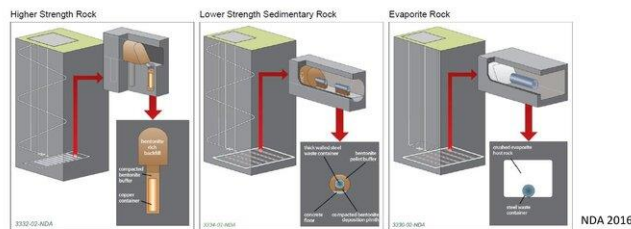
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UNCERTAINTY MANAGEMENT AS THE DISPOSAL PROGRAMME PROGRESSES

- Examples of safety assessment approaches in early programme phases
 - Safety assessments may be undertaken in the context of a range of different disposal concept options where, for example, decisions are yet to be made on waste packaging options or the type and location of host rock for the facility (e.g., Netherlands, UK)
 - enables key parameters associated with disposal system performance (and uncertainties in those parameters) to be identified, albeit in the context of credible generic assumptions about natural and engineered barrier system characteristics



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UNCERTAINTY MANAGEMENT AS THE DISPOSAL PROGRAMME PROGRESSES

- Examples of safety assessment approaches in early programme phases
 - safety assessments may be undertaken in support of site evaluations as part of a site selection process
 - supports decisions about the feasibility of disposal and identifies key uncertainties to be investigated in the next phase of site characterisation (e.g. through intrusive investigations) and RD&D (UK)
 - preparatory safety assessments may done in parallel with system development (Belgium)
 - used repeatedly to determine the relevance of uncertainties to safety and provide guidance on RD&D activities

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MANAGING EMERGING UNCERTAINTIES

- Some uncertainties will decrease as new information becomes available (e.g., ‘as-built’ properties, monitoring data, and RD&D results), but activities associated with the programme (process modelling, safety assessment, siting, and construction) can lead to new viewpoints and sometimes new uncertainties (i.e. **emerging uncertainties**)
- Issues associated with emerging uncertainties can be dealt with, mitigated or guarded against by for example
 - verification of assumptions during each programme phase through RD&D, characterisation, monitoring and inspection activities, and feedback from construction and operational phase observations
 - learning from international experience
 - use of proven methods and materials
 - implementation of quality management system principles (QA/QC) in the various activities carried out in the programme (modelling, design, construction)
 - the mitigation of programme uncertainties and associated risks (e.g., financial resources, stakeholder conditions, available knowledge and skill)
 - implementation of the defence in depth principle

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MANAGING EMERGING UNCERTAINTIES

- Implementation of the **defence in depth principle** may include
 - implementation of a **multi-barrier disposal system** that provides independent and complementary safety functions and components, ensuring sufficient safety is maintained following occurrence of an uncertain disruptive event or process
 - measures to **prevent deviations** from normal operation and the failure of items important to safety (such as when subjected to disturbances)
 - use of safety margins and reduction or avoidance of sources of uncertainty having the potential to jeopardise safety (e.g., by modifying the location or design of the repository)
 - preservation of knowledge about the repository
 - quality control measures and procedures to prevent defects and damage to disposal system components
 - implement measures to **detect deviations** from normal operation, defects or damage to the disposal system so that corrective measures can be taken where needed (e.g., waste retrieval where allowed for)
 - measures to **prevent or mitigate the consequences** of accidents that would result from failure of other measures

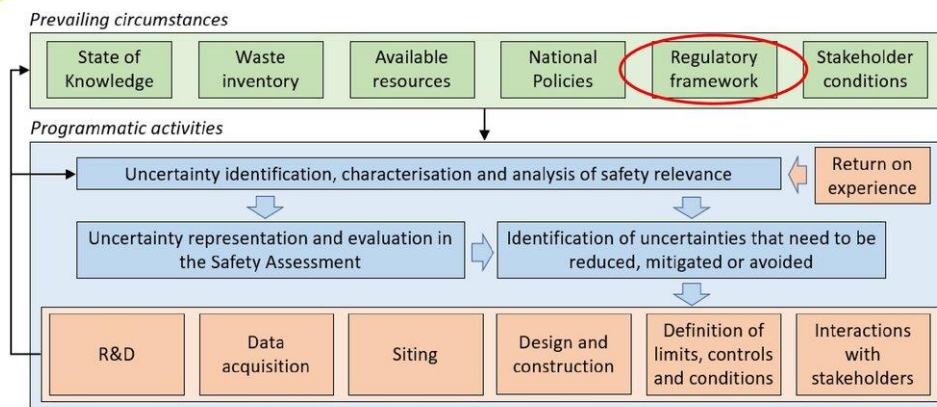
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ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY



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REGULATORY FRAMEWORK

- Regulatory requirements cover a range of safety standards that need to be addressed in the safety case
 - radiological dose and/or radiological risk constraints
 - protection of non-human organisms and the accessible environment
 - protection against non-radiological hazards
 - groundwater protection
- Assessments against these requirements will influence
 - the safety functions to be provided by different components of the barrier system
 - disposal system optimisation
- Regulatory guidance on how to address these requirements influences how uncertainty is addressed in the safety demonstration

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REGULATORY FRAMEWORK

- Examples of how radiological dose or risk constraints may vary to reflect uncertainties associated with long disposal system evolution **timescales** and **unlikely scenarios**
- Such requirements ensure that assessments of unlikely scenarios or scenarios over very long timescales do not unduly influence design optimisation

Date

	Constraint	Conditions
General	Dose: 0.1 to 0.5 mSv/year	Typical post-closure dose constraint for members of the public
	Risk: 10 ⁻⁶ per year	Typical annual risk constraint for an individual member of a potentially exposed group
France	Dose: 0.25 mSv/year	Reference conditions (normal evolution) on a timescale of at least 10,000 years
	Calculated dose levels should not be unacceptable (0.25 mSv/year is used as a reference level)	Reference conditions on timescales greater than 10,000 years
	Calculated dose should be low compared to levels that could lead to deterministic effects	Altered evolution scenarios
Belgium	Dose: 0.1 mSv/year	Expected evolution scenario
	Dose: 0.3 mSv/year	Reference scenarios (conservative assumptions)
	Dose: 3 mSv/year	'Penalising' scenarios (stylised worst-case illustrations)
	Risk: 10 ⁻⁶ per year	Expected evolution scenario
	Risk: 10 ⁻⁵ per year	Altered evolution scenarios
Slovenia	0.3 mSv/year	normal evolution scenario
	Calculated dose: < 10 mSv/year	Altered evolution scenarios for which no optimisation is required
	Calculated dose: 10 - 100 mSv/year	Altered evolution scenarios for which measures to minimise scenario probability are required
	Calculated dose: > 100 mSv/year	Altered evolution scenarios for which measures to minimise scenario consequences are required
UK	Dose calculation supports design optimisation	Human intrusion scenarios

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REGULATORY FRAMEWORK

- Example of requirements on post-closure criticality scenario assessment, where regulations generally require a demonstration that criticality is not possible during transport and disposal of fissile wastes, which is achieved in part through packaging constraints, but requirements relating to post-closure criticality differ
 - it may be necessary to show that post-closure criticality is not a significant concern (UK)
 - a post-closure scenario assessment would aim to show that criticality is unlikely and, if it did occur, it would be of low consequence to repository performance
 - the unlikely post-closure criticality scenario has limited influence on design optimisation
 - alternatively, it may be necessary to show that appropriate measure have been taken to ensure criticality could not occur after repository closure (Switzerland)
 - a post-closure scenario assessment would need to show that criticality would not occur, with uncertainties in waste package evolution over long timescales taken into account
 - uncertainty about waste package evolution over long timescales may greatly influence waste package design, with defence in depth in design being important to eliminate the possibility of criticality occurring

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REGULATORY FRAMEWORK

- Other types of regulatory requirements or guidance relating to uncertainty treatment
 - decisions about implementing uncertainty avoidance, reduction or mitigation options should be made in the context of the overall objective of ensuring that the detrimental impacts of disposal are as low as is reasonably achievable (ALARA) (i.e. with reference to the principle of **optimisation** and the use of Best Available Techniques (BAT))
 - there is adequate **defence in depth** (which may be part of an uncertainty management strategy), so that there is a combination of several layers of protection (e.g., safety functions provided by physical barriers, systems to protect the barriers, and administrative procedures) that would have to fail or be bypassed before there could be any consequences for people or the environment
 - it may be required that wastes remain **retrievable** after disposal, at least for a certain period of time, which may mitigate concerns that, ultimately, it may not be possible to reduce uncertainties sufficiently to satisfy all safety requirements
 - uncertainty may increase if the disposal system design becomes more complicated
 - long-term safety should not be impacted by measures that allow for retrievability

Date

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REGULATORY FRAMEWORK

- Other types of regulatory requirements or guidance relating to uncertainty treatment (cont.)
 - **all uncertainties significant to safety** should be identified and adequately taken into account in the safety case
 - a scenario analysis should be included that considers all **features, events and processes** (FEPs) that might affect the performance of the disposal system
 - a programme of **uncertainty management** should be presented in the safety case
 - a **register of significant uncertainties** should be maintained, informed by safety assessment calculations, which will inform future phases of disposal system development (e.g., site characterisation activities and design modifications) and future research needs, with the aim of focusing on reduction or mitigation of the key uncertainties

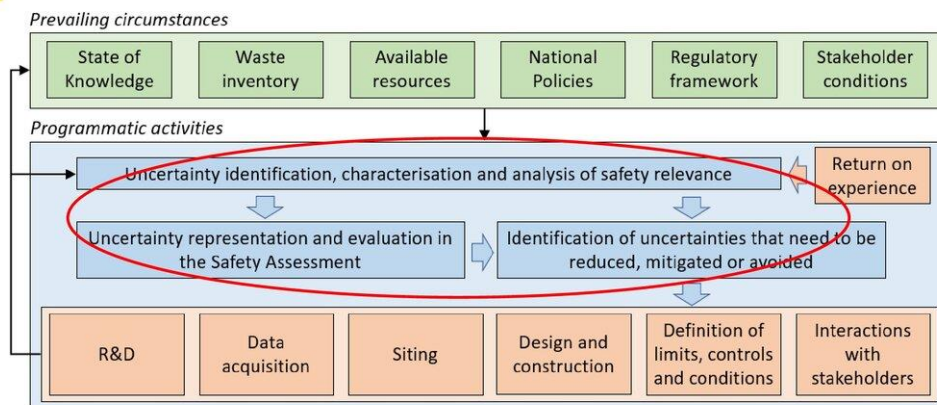
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ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY



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TYPES OF UNCERTAINTY

- Uncertainties relevant to the safety assessment have to be identified and characterised with respect to their source, nature and degree, using quantitative methods, professional judgment or both
 - analysis of FEPs that could affect disposal system behaviour commonly forms part of the uncertainty identification process
- In general, the following types of uncertainty need to be managed in a disposal programme and their relevance to safety assessed
 - programme uncertainties
 - uncertainties associated with the initial characteristics of the waste, site and engineered components
 - uncertainties in the evolution of the disposal system and its environment, including the effects of events and processes that may affect the initial characteristics of the disposal facility (e.g., hazards that may occur during construction and operation)
 - uncertainties associated with the data/parameters, models and methods used in the safety case
 - uncertainties associated with the completeness of the FEPs considered in the safety case

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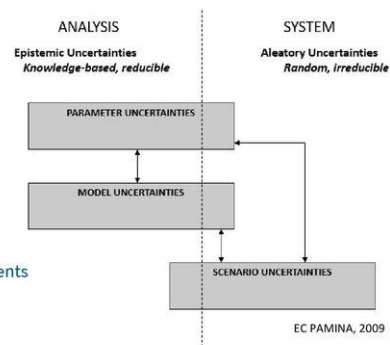
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TYPES OF UNCERTAINTY

- These uncertainties can be classed as
 - epistemic uncertainties
 - relate to knowledge or the degree of its validation
 - e.g. uncertainty about site characteristics
 - can be reduced
 - aleatory uncertainties
 - relate to random events
 - e.g. uncertainty over the time of occurrence or magnitude of rare events
 - cannot be reduced



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TREATMENT OF UNCERTAINTY IN SAFETY ASSESSMENTS

- Uncertainties over future states of the disposal system are typically addressed in a safety assessment by considering different scenarios of disposal system evolution (**scenario uncertainties**)
- **Data/parameter and model uncertainties** associated with each scenario can be addressed by
 - demonstrating that the uncertainty is irrelevant or of low consequence to safety
 - representing the uncertainty explicitly using, for example, probabilistic techniques
 - bounding the uncertainty and showing that the bounding case gives acceptable safety
 - taking a stylised approach to handling uncertainty where few relevant data are available or can be gathered, which involves making plausible assumptions that tend to err on the side of conservatism
 - testing sensitivity to alternative credible models of uncertain processes
 - ruling out an uncertain but potentially significant FEP on the grounds of very low probability of occurrence, or because other consequences, were the FEP to occur, would far outweigh concerns over disposal facility performance
- **Vinzenz will discuss different methods for identifying, categorising and characterising these types of uncertainty, and ranking their importance to safety assessments**

Date

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TREATMENT OF UNCERTAINTY IN SAFETY ASSESSMENTS

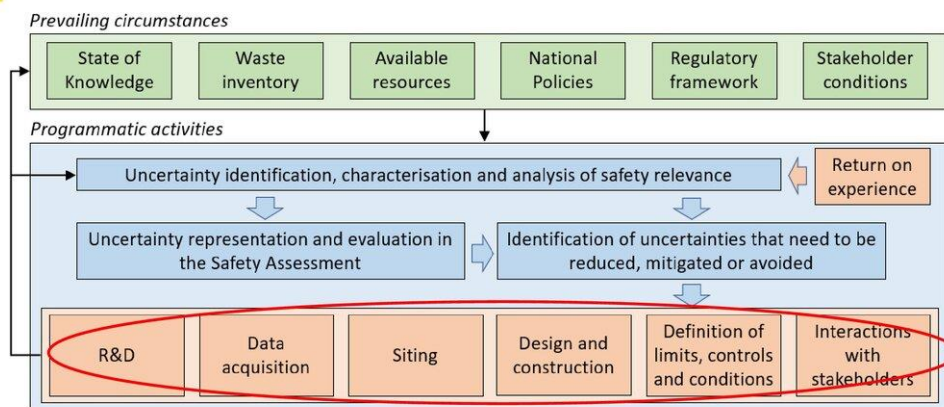
- Assessing the impact of uncertainties is done through
 - **sensitivity analysis**, which looks at how calculated indicators of safety are influenced by uncertainty in input parameters
 - **uncertainty analysis**, which looks at uncertainty in calculated indicators of safety
- **Klaus will discuss approaches to sensitivity and uncertainty analysis**

Date

Training material



ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY



Date

Training material





AVOIDANCE, MITIGATION AND REDUCTION OF UNCERTAINTIES

- Strategies for addressing uncertainties will involve a mixture of reduction, avoidance and mitigation that will adapt as programmes progress
 - further data acquisition
 - RD&D, although this is not likely to reduce unquantifiable uncertainties, particularly those relating to the very long timescales considered by assessments
 - constraining the inventory
 - characterisation or processing of waste
 - site selection and site characterisation
 - adapting the disposal concept
 - adopting particular construction, operation and closure methods (use of BAT)
 - maintaining/updating the management system (e.g., reduction of uncertainties on as-built properties through QA/QC measures)
 - allowing for retrieval in the event that it is not possible to reduce uncertainties sufficiently to satisfy all safety requirements

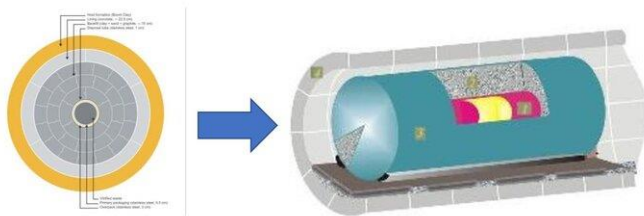
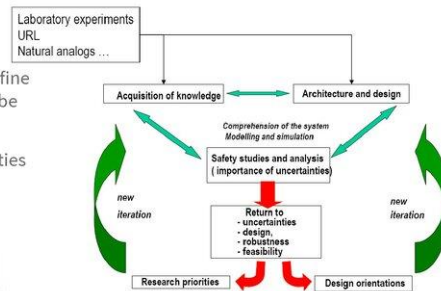
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AVOIDANCE, MITIGATION AND REDUCTION OF UNCERTAINTIES

- Examples of approaches to uncertainty reduction and avoidance
 - successive safety assessments in the French programme were used to refine site investigations and focus on those features of the host rock found to be most significant to its performance as a barrier
 - the Belgian ‘supercontainer’ concept was introduced to avoid uncertainties concerning near-field corrosion processes (amongst other things)



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Training material





MANAGING DISPOSAL DIFFERENT TYPES OF UNCERTAINTY

- Next will discuss options for managing
 - programme uncertainties
 - inventory uncertainties
 - disposal concept uncertainties
 - uncertainties in as-built conditions
 - uncertainties in disposal system evolution
 - uncertainties in data and models
 - uncertainties in the completeness of FEP and scenario analysis

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Training material



MANAGING DISPOSAL PROGRAMME UNCERTAINTIES

- Programme uncertainties include those related to
 - decisions on future energy policy (i.e., nuclear versus other technologies)
 - decisions on waste management practice (e.g., storage versus disposal, disposal of spent nuclear fuel versus reprocessing).
 - timing of repository programme decisions and their implementation (siting, construction, operation, and closure of facilities)
 - the regulatory framework
 - schedule of waste arisings
 - available financial resources, skills and experience

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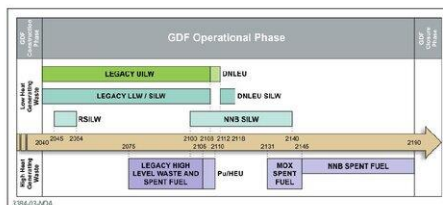
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MANAGING DISPOSAL PROGRAMME UNCERTAINTIES

- Addressing programme uncertainties
 - mitigation measures to address uncertainties in timing of programme decisions include stakeholder communication and involvement, review of strategy, monitoring of costs and allocation of resources, knowledge management, maintenance of technical skills, and ensuring political awareness of the long-term necessity of sufficient resources and maintenance of competences
 - in the early stages of a disposal programme, uncertainties may be managed through 'planning' assumptions about when facilities will become available, where facilities might be located, and what wastes will arise



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- in later stages, uncertainties are related to facility closure and the possible need for remedial action in light of developing waste management practices

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MANAGING DISPOSAL PROGRAMME UNCERTAINTIES

- Programme uncertainties may be addressed in cost estimations, which consider
 - uncertainties related to the maturity of the technologies used (e.g. innovative technologies) (e.g. technology contingencies in Slovenia programme).
 - uncertainties related to the maturity of the project, which include omissions and unforeseen costs caused by an incomplete definition of the project and its engineering (e.g. project contingencies in the Slovenian programme).
- Programme uncertainties may be managed by basing techniques for repository construction, operation and closure (and their costs) on available and proven technologies (BAT), and only adopting new technologies when they become sufficiently mature (e.g., Belgium, Netherlands)
- Socio-economic costs may be managed by acquiring finances for the facility upfront and the facility developer becoming the owner of the waste, which mitigates uncertainties relating to financial resources and the long-term existence of organisations that have generated the waste (Netherlands)

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MANAGING INVENTORY UNCERTAINTIES

- Programme uncertainties can lead to uncertainties in the waste inventory, wasteforms for disposal and waste packaging options
 - address by considering alternative inventories in the safety assessment or cost estimate
 - adopt disposal concept that can be readily extended to accommodate additional waste volumes using (Netherlands)
- Uncertainties in estimates of waste arisings occur because of uncertainties in waste characterisation, waste conditioning, waste packaging, and the disposal programme schedule
 - regular inventory tracking and reporting - inventory uncertainties reduce as programmes evolve and the wastes are actually packaged and emplaced
 - use statistical analysis of historical data collected from nuclear power plants or other relevant facilities in operation
 - use upper bounding values or conservative scaling factors for inventory parameters
 - develop waste acceptance criteria (WAC) derived from the safety assessment to ensure waste packages conform to a standard
 - impose controls during waste acceptance to minimise possibility of emplacing waste packages that would not meet the WAC

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Training material



MANAGING DISPOSAL CONCEPT UNCERTAINTIES

- Disposal concept uncertainty is greatest in the early programme phase and reflects decisions to be made about the wastes for disposal and the type of disposal facility
 - illustrative designs for the disposal of different categories of waste in different types of host rock may be assessed to provide demonstrations of how safety cases could be made (e.g. UK)
 - design variants (e.g. drift vs. shaft) may be assessed (e.g. Switzerland)
 - robustness of disposal concepts may be assessed through sensitivity analyses, which may account for human factors such as defects in design or manufacture, human error, etc. (France, Switzerland)
- Once the type of facility and the host rock have been decided, disposal concept uncertainties will reduce

TRU waste package disposal in vaults and test emplacement of borehole disposal in evaporite at WIPP, USA



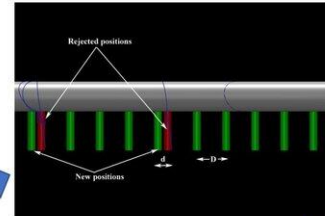
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MANAGING UNCERTAINTIES IN AS-BUILT CONDITIONS

- A safety assessment must address the feasibility and reliability of proposed construction methods and the technical feasibility of the proposed design options
 - where there are large uncertainties, such as in the use of novel techniques, design alternatives may need to be considered based on technical options that have been demonstrated to be feasible
 - analysis of potential material QC or barrier emplacement errors, or undetected accidents, may be used to define conservative assumptions or alternative disposal system evolution scenarios based on different assumptions about conditions at the time of facility closure (Belgium, Sweden, UK)
 - feedback from experience during construction, and the results of maintenance, periodic testing, and inspection, will support uncertainty reduction
 - QA/QC activities during repository construction, operation and closure will help to reduce uncertainties in the as-built state



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MANAGING UNCERTAINTIES IN DISPOSAL SYSTEM EVOLUTION

- Safety assessments generally address uncertainties in long-term evolution as follows
 - uncertainties in parameter values may be captured in parameter value distributions for use in probabilistic assessments, or upper and lower values of a range for use in deterministic assessments
 - uncertainties in barrier performance may be captured by defining different scenarios of disposal system evolution based on different assumptions about how the barriers provide safety functions when subject to the effects of uncertain events or processes
 - uncertainties in conditions in the very long-term may be treated by defining different scenarios of disposal system evolution (such as associated with different assumptions about climate change and its impacts or seismic events) and assessing those scenarios deterministically

Period	Illustration	State & Notes
250 to 310 ka		Boreal: Further incision in the upper catchment and aggregation of coarser deposits in the valley floor; continued isostatic adjustment.
310 to 335 ka		Periglacial: Further incision and aggregation of coarser sediments; continued isostatic adjustment; likely to be extensive hunting and herding activities.
335 to 354 ka		Glacial: Potential for glacial scouring, along with further incision and aggregation of coarser sediments; continued isostatic adjustment; unlikely to be permanent human occupancy.
354 to 357 ka		Periglacial: Further incision and aggregation of coarser sediments; continued isostatic adjustment; likely to be extensive hunting and herding activities.
357 to 360 ka		Boreal: Further incision in the upper catchment and aggregation of coarser deposits in the valley floor; continued isostatic adjustment.
360 to 370 ka		Temperate: Return to gradual denudation; finer sediments accumulate over the coarser deposits in the valley floor; gradual increase in catchment size.

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MANAGING UNCERTAINTIES IN DISPOSAL SYSTEM EVOLUTION

- Scenarios of disposal system evolution
 - typically a base or reference scenario is assessed which describes the features of the disposal system at closure and the way in which that system is expected to evolve, taking account of significant quantifiable uncertainties
 - deviations from the base scenario caused by FEPs that may or may not occur are considered as variant scenarios
 - human activities relevant to long-term safety assessments are always likely to be highly uncertain and are treated as variant or 'what-if' scenarios
- Site characterisation and material research will reduce quantifiable uncertainty in the long-term performance of natural and engineered barrier systems

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MANAGING UNCERTAINTIES IN DISPOSAL SYSTEM EVOLUTION

- An analysis of system uncertainties can be used to define a timescale for probabilistic calculations; that is a timescale during which all significant uncertainties can reasonably be represented explicitly
- For uncertainties that cannot readily be quantified, consider the timescale on which these become more significant than those that can be quantified
 - when cycles of major climate change (including glaciations) become possible, significant unquantifiable uncertainties relating to the evolution of a disposal system exist
 - beyond this timescale, deterministic 'what-if' calculations and reasoned qualitative arguments might be preferred over probabilistic analysis to illustrate possible system performance

GDF closure	100 years	1,000 years	10,000 years	100,000 years	1,000,000 years
Transient changes	Stable conditions		Biosphere and geosphere evolution Increasing uncertainty in system behaviour		
Narrative of disposal system evolution based on understanding of wasteform and barrier behaviour, supported by simple insight modelling					
Environmental safety arguments based on natural and archaeological analogue evidence					
Probabilistic total system model calculations of radionuclide migration and risk					
Deterministic modelling of the effects of natural processes on GDF performance					

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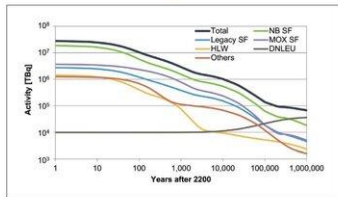
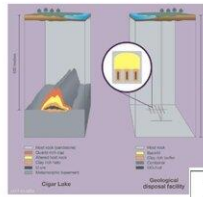
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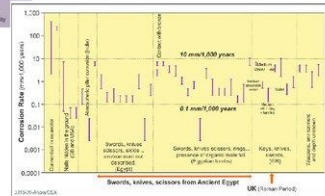
MANAGING UNCERTAINTIES IN DISPOSAL SYSTEM EVOLUTION

- Qualitative arguments regarding uncertainties in long-term evolution might involve
 - consideration of natural analogues and anthropogenic evidence
 - balancing increasing uncertainty with time against a decrease in radiotoxicity



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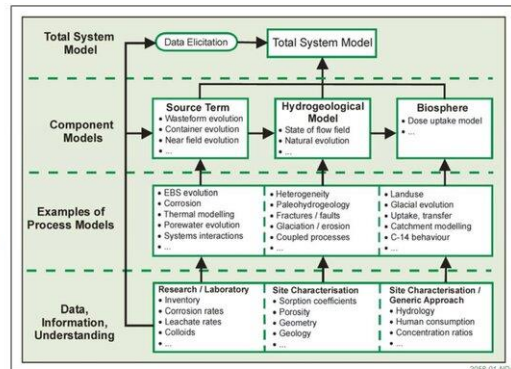


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MANAGING UNCERTAINTIES IN DATA AND MODELS

- How data are stored and recorded is relevant to the management of uncertainties
 - a data management system is important to ensuring the quality of data at all levels in safety assessment models
 - will need to store metadata, including provenance and sources of uncertainty
 - there are likely to be hierarchies of models through which data and information about uncertainties need to be propagated



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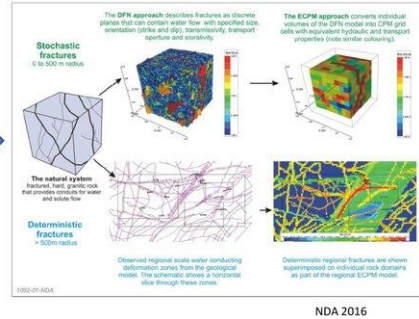


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MANAGING UNCERTAINTIES IN DATA AND MODELS

- Model uncertainty is the uncertainty in the calculation models used in the safety assessment
 - where there is model uncertainty, assumptions can be made that overestimate the consequences of unfavourable processes and conversely that underestimate or neglect the potentially positive consequences of favourable processes
 - alternative models or alternative approaches to simplification may be used to illustrate uncertainties caused by model simplifications
- Verification and, as far as possible, validation of models and computer codes used in safety assessments supports avoidance or minimisation of model uncertainties
 - validation usually involves comparing the results of specific process simulations with experimental data, field observations, and/or natural analogues



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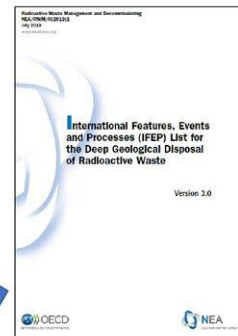
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MANAGING UNCERTAINTIES IN COMPLETENESS OF FEP AND SCENARIO ANALYSIS

- The scope of FEP identification and analysis needs to be sufficient to meet the needs of the safety assessment; uncertainty in the completeness of FEP analysis can be reduced through
 - the use of well-understood and compatible materials for the engineered barriers and the selection of a host rock that is unlikely to be affected in the future by any unrecognised natural phenomenon or future human action
 - the application of a FEP management process throughout the disposal programme
 - the use of safety margins in the design of engineered repository components (e.g., cautious waste container thickness)
 - participation in or reference to international projects and FEP analysis done by other organisations



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MANAGING UNCERTAINTIES IN COMPLETENESS OF FEP AND SCENARIO ANALYSIS

- A safety case is likely to include a FEP analysis with a justification of why FEPs are discarded from scenario development or how they have been treated
 - the scenario development methodology should provide sufficient confidence in the completeness of the scenarios identified and, hence, minimise uncertainties associated with the completeness of the FEPs considered
- altered evolution scenarios and what-if scenarios assuming the loss of a barrier or a safety function can be used to assess the robustness and defence-in-depth provided by the disposal system; such an approach substantiates that some degree of uncertainty in the completeness of FEPs would not jeopardise safety

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SUMMARY AND CONCLUSIONS.

- This lecture has explained the importance of implementing an uncertainty management strategy in the early stages of a disposal programme
 - information about uncertainties informs decisions to be taken at any programme step
 - how uncertainties are accepted and managed evolves throughout programme phases, and is influenced by factors such as
 - the state of knowledge
 - the regulatory framework
 - various methods may be used to avoid, mitigate or reduce uncertainties and a description of the programme for uncertainty management needs to be included in a safety case
- The various types of uncertainty to be addressed in a safety case have been described and options for managing them have been summarised (detailed approaches to the treatment of different types of uncertainty in safety assessments are discussed in later lectures)
- A well managed uncertainty management strategy is an essential part of a programme that will lead to implementation of an optimum solution for the long-term safety of radioactive waste disposal

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Appendix C. Slides of lecture A.4

A.4 Uncertainty identification and quantification

Prepared by Vincenz Brendler, HZDR



A.4 UNCERTAINTY IDENTIFICATION AND QUANTIFICATION

Vincenz Brendler, HZDR • WP 10



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14.02.2023

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KEY WORDS

- Numerical uncertainties
- Uncertainty models
- Methodologies
- Categorization
- Characterization
- Treatment
- Evaluation

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CONTENTS

- **Learning outcomes**
- **Introduction:**
 - Why are uncertainties that important & how you can benefit from the lecture
 - UMAN – a little bit of history
 - Selected Glossary terms
 - Major Goal →Draft a Methodology
- **Identification:**
 - Relevance criteria
 - Bottom-Up vs. Top-Down
 - Survey methods for established approaches
 - Questionnaire – How to include a variety of opinions & approaches
- **Uncertainty characterization:**
 - Examples from geochemistry, geology and long-term stability
- **Uncertainty categorization:**
 - Alternative categorization Types
 - Uncertainty categorization specific to EURAD
- **Uncertainty quantification & Numerical evaluation**
- **Summary & Conclusions**
- **Bibliography**

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LEARNING OUTCOMES

After the completion of this training unit/lesson:

- 1** Students/participants should be able to understand the major elements and internal logics of uncertainty identification focussed on numerical parameters
- 2** Participants can apply important definitions linked to uncertainty treatments
- 3** Different ways of uncertainty categorization and presentation are known

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INTRODUCTION – WHY ARE UNCERTAINTIES THAT IMPORTANT?



Rui Zink – Die Installation der Angst | lustauflesen.de

- Decisions associated with Radioactive Waste Management (RWM) programs are made in the presence of irreducible and reducible uncertainties.
=> Uncertainty creates an uncomfortable position for a large part of the public (anxiety).
- Uncertainties are a **big challenge** (known unknowns, unknown unknowns etc.; difficulties to express them numerically, multi-dimensional problem, variety of mathematical approaches, scenario-dependent, ...)

Identification \ Certainty		Certain (Known)	Uncertain (Unknown)	
			Impact	Occurrence
Identified (Known)		Known known (identified knowledge)	Known unknown (identified risk)	
Unidentified (Unknown)	Consequence	Unknown known (untapped knowledge)	Unknown unknown (unidentified risk)	
	Event			

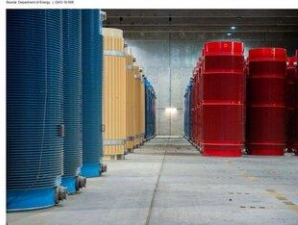
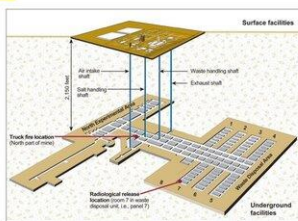
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INTRODUCTION – HOW YOU CAN BENEFIT FROM THE LECTURE



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- Develop a common understanding among the different categories of actors on uncertainty management and how it relates to risk & safety. In cases where a common understanding is beyond reach, the objective is to achieve mutual understanding on why views on uncertainties and their management are different for different actors.
- Share knowledge/know-how and discuss common methodological/strategical challenging issues on uncertainty management.
- Identify the contribution of past & on-going RD&D projects to the overall management of uncertainties.
- Identify remaining and emerging issues and needs associated with uncertainty management.



UMAN – A LITTLE BIT OF HISTORY ...

- Subtask 2.2 (ST 2.2) “Uncertainty identification, classification and quantification” started work in June 2019.
- Team of ST 2.2 is composed of partners from:
CIEMAT, EIMV, GRS, HZDR, Nagra, NDA, SCK-CEN, STUBA, SURAO, TU Sofia
- A questionnaire was developed together with ST 2.3 “Methodological approaches to uncertainty and sensitivity analysis” to ask for established knowledge on:
 - how to identify numerical uncertainties (i.e. uncertainties concerning quantities, sometimes also called parameter uncertainties) that might be of relevance
 - which possible schemes of classification of uncertainties are utilized so far
- Together with my colleague *Solveig Pospiech* and substantial help from various ST 2.2 members (plus *Andra, IRSN, TUL, UDC & UJV*) a report was drafted, with its final version under external review since December 2022.

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SELECTED GLOSSARY TERMS (MAY ALSO BE HELPFUL FOR FOLLOWING LECTURES)

- **Aleatory Uncertainty:** The stochastic part of the uncertainty of an input parameter that forms an intrinsic property of the parameter and that cannot be reduced. An aleatory random variable represents the possible outcome of an observation of the quantity.
- **Epistemic Uncertainty:** The part of the uncertainty of an input parameter resulting from limited knowledge of the natural conditions and processes that can in principle be reduced by obtaining more information. An epistemic random variable represents the state of knowledge about the quantity.
- **Features, Events, and Processes (FEP):** These are terms used in the fields of radioactive waste management, carbon capture and storage, and hydraulic fracturing to define relevant scenarios for safety assessment studies.
- **Geological Domain:** A spatial distinct region or subregion in the geological formation with similar modal mineral composition, structural properties, spatial orientation and anisotropy, rock density, porosity and rock mechanic properties.

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SELECTED GLOSSARY TERMS (CONTD.)

- **Goal Breakdown Structure (GBS):** The EURAD goals breakdown structure is a thematic breakdown of knowledge and generic activities essential for radioactive waste management. It comprises Themes (Level 1), Subthemes (Level 2) and Domains (Level 3), each formulated as goals. *Although hierarchical and numbered, the knowledge and activities presented across the GBS should be considered collectively with no weighting to order of importance. Rather it is emphasised that there are many inter-dependencies and linked data across the GBS, where knowledge and activities can be centred in different ways, depending on the end user role and precise boundary conditions of a specific RWM programme.*
- **Sensitivity Analysis (SA):** The process of appreciating the dependency of the model output from model input. It also investigates how important each model input is in determining the output.
- **Uncertainty:** Lack of objective information (evidence) or subjective information (knowledge).
- **Uncertainty Analysis (UA):** The process of exploring the uncertainty in the model output.

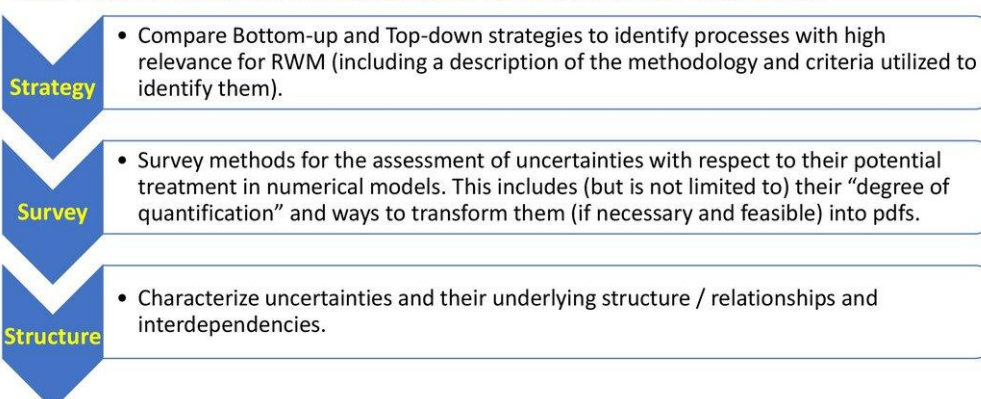
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MAJOR GOAL: DRAFT FIRST STEPS FOR A METHODOLOGY FOR UNCERTAINTY TREATMENT



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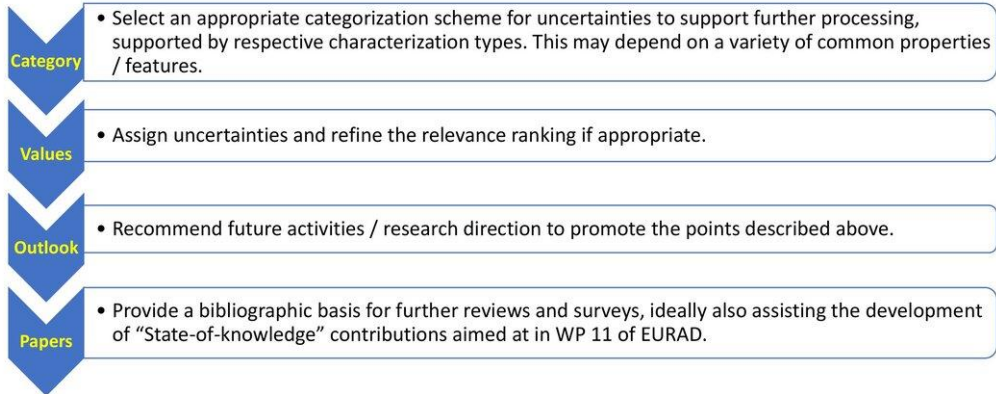
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**MAJOR GOAL:
DRAFT FIRST STEPS FOR A METHODOLOGY FOR UNCERTAINTY TREATMENT**



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IDENTIFICATION OF RELEVANCE CRITERIA

Checks for relevance are essential as the sheer number of uncertainties is overwhelming ...

Relevance can be derived from:

- Level of impact on safety
- Level of impact on decision-making process
- Priority for further investigation
- Cost-benefit-ratio

		Uncertainty		
		Large Amount of Knowledge	Medium Amount of Knowledge	Low Amount of Knowledge
Severity	Catastrophic Impact	Critical	Critical	Critical
	Medium Impact	Potential	Potential	Potential
	Low Impact	Non-Critical	Non-Critical	Potential

Problem: such criteria are obviously regularly used but seldom specified !?

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IDENTIFICATION: BOTTOM-UP VS. TOP-DOWN

Bottom-up (BU) and **top-down (TD)** modelling strategies are utilized for many complex application cases in science and society beyond RWM. It is advised to have understanding of both philosophies, their strengths and weaknesses.

BU: builds on detailed knowledge and process understanding on a mechanistic level, which fosters public acceptance of specific safety cases. But typically it has an enormous amount of details (200+ parameters collected by the OECD/NEA Crystalline Club alone for assessing host rock properties in the safety case) => hard to parameterize, requires huge amount of computing time. On the other side, it allows many parameters to be declared insensitive already at an early stage of model development.

TD: focuses on integration of system components & can handle large numbers of uncertain parameters often easier, but may overlook higher-order effects and not cover all regions of interest.

➤ **Combine them:** BU approaches can provide generic parameters requested by TD models that usually start on a rather coarse level and will then be iteratively refined.

A **hierarchy of models** (often to be refined iteratively) can be required for complex systems. Depending on the specific application field within RWM, the mutual relationships between BU and TD and their respective weight may vary. BU models clearly scale with the dimensionality.

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SURVEY METHODS UTILIZING ESTABLISHED APPROACHES

Description of sources used for to identify and rank uncertainties:

- Compilation from expert elicitation, here primarily based on a respective **questionnaire** send out in 2021 to UMAN ST 2.2 & ST2.3 participants
- Interactions with UMAN Task 3 teams, utilizing their reports
- Evaluation of a literature survey (with feedback from 14 UMAN partners)



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QUESTIONNAIRE – INCLUDE A BROAD VARIETY OF OPINIONS & APPROACHES

- 1) Which category of numerical uncertainties do you typically encounter in your work? Please label your answers whether they are aleatory or epistemic.
- 2) Which rules / handbooks / best practices / ... are used in your work group to treat uncertainties?
- 3) How do you identify the relevance of uncertainties?
- 4) Which numerical parameters are in the focus of your experimental or modelling work, to which processes / phenomena are they related?
- 5) Describe the types of uncertainties relevant for the example.
- 6) By which means did you quantify these uncertainties?
- 7) How do you parameterize the uncertainties?
- 8) Which methods are used to verify *post mortem* a correct assignment of approach and parameterization?

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UNCERTAINTY CHARACTERIZATION

- Advices on how uncertainties can be **characterized, illustrated and structured** to pave the way to a comprehensive assessment of numerical uncertainties:
 - Fishbone (Ishikawa) diagrams
 - Uncertainty table
 - Verbal description (see Tim Hicks about Managing Disposal Programme Uncertainties slides #11-14 - and others)
- Representative examples will follow for uncertainty hierarchies & interdependencies (already grouped according to the EURAD GBS)
- **Specifics** for the **Bottom-up** approach:
 - Basic uncertainties will effect “very different processes very differently”
 - Uncertainties may be different for alternative models (for the same process)
 - There is a hierarchy of models, up-scaling is a serious challenge calling for model reduction (+ ML applications)

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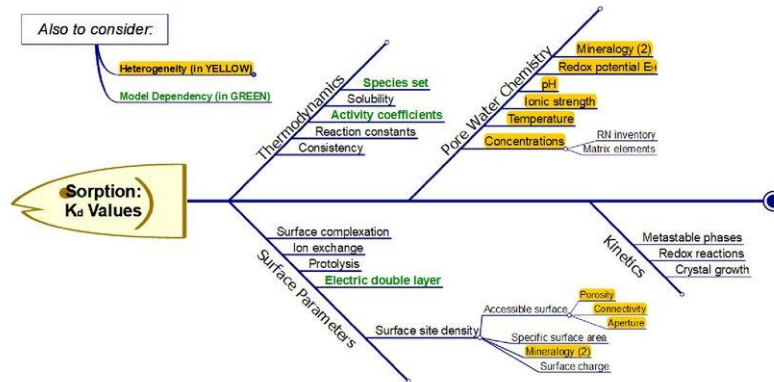
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UNCERTAINTY CHARACTERIZATION: GEOCHEMISTRY EXAMPLE #1



Fishbone (Ishikawa) diagram, highlighting the mutual relationship between uncertainties contributing to the overall uncertainty of distribution coefficients describing sorption.

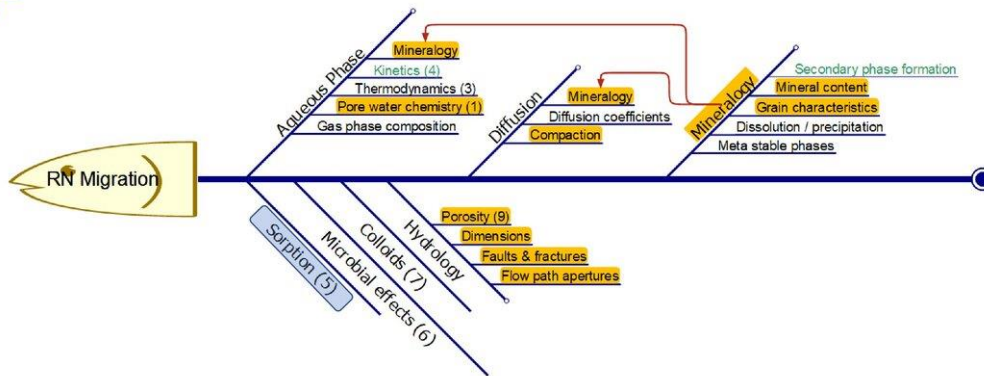
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UNCERTAINTY CHARACTERIZATION: GEOCHEMISTRY EXAMPLE #2



Fishbone (Ishikawa) diagram analyzing radionuclide migration (with sorption being one [important] submodel).

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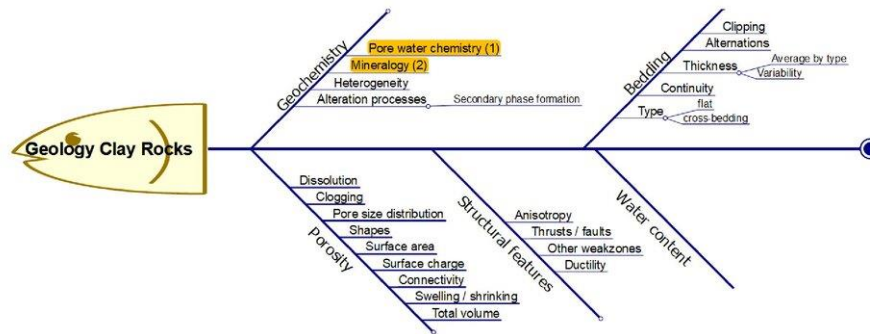
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UNCERTAINTY CHARACTERIZATION: GEOLOGY EXAMPLE #1



Fishbone (Ishikawa) diagram illustrating the uncertainties related to argillaceous host rocks.

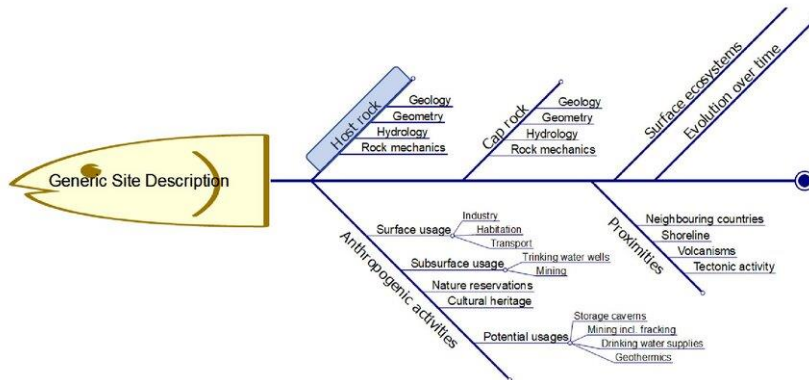
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UNCERTAINTY CHARACTERIZATION: GEOLOGY EXAMPLE #2



Fishbone (Ishikawa) diagram describing uncertainties for a generic DGR site.

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UNCERTAINTY CHARACTERIZATION: LONG-TERM STABILITY EXAMPLE #1

- Uncertainty table describing the hierarchy of contributions to the overall uncertainty associated with the geological and tectonic evolution of a DGR.

EURAD Subtheme	EURAD Domain	Induced effects	Associated uncertainties	Epistemic / Aleatoric
Long-term stability	Geological and tectonic evolution (Seismicity & Faulting)	changes in hydrogeology (seismic pumping)	frequency; amplitude; time of occurrence	A
		faults growth	fault size	A
		new faults creation --> changes in the water field and transport	fault size and permeability, geometry of aquifers (thickness, depth and extent)	A
		new fractures	hydraulic properties	E

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UNCERTAINTY CHARACTERIZATION: LONG-TERM STABILITY EXAMPLE #2

- Climate uncertainty table.

EURAD Subtheme	EURAD Domain	Induced effects	Associated uncertainties
Long-term stability	Climate (general and extreme conditions)	Cooling (up to permafrost) or warm-up: porosity changes	Temperature shift, water table level; time interval of wet periods, permafrost depth
		wet periods: changing infiltration and recharge rates; water table level; groundwater chemistry	Infiltration / evapotranspiration rate; time interval of wet periods
		desertification: increased soil erosion	erosion rate
		shoreline displacement	time of occurrence, extent of displacement

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UNCERTAINTY CHARACTERIZATION: LONG-TERM STABILITY EXAMPLE #3

- Glaciation uncertainty table.

EURAD Subtheme	EURAD Domain	Induced effects	Associated uncertainties
Long-term stability	Climate (Glaciation)	interglacials: large volumes of surface water (ice melts) affects topology, ground water fluxes, flow directions	interglacial cycle & their numbers; stress fields; flow parameters; water chemistry; temperature
		glacial periods: effects on the surface environment and groundwater	glacial cycle/amplitude; permafrost depth; ice thickness; stress fields; flow parameters; temperature
		repeated glaciations: may exhume the repository	erosion rate
		blockage of pathway due to precipitation and filtration of colloids and particles	flow field

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UNCERTAINTY CATEGORIZATION

- Categorization supports the identification of uncertainties with high relevance for RWM:
 - Provides a systematic and uniform approach to describe uncertainties
 - Gives hints on how to manage them
 - Provides hierarchical structures and mutual dependencies
- The internet is full of respective **taxonomies** – but they are either too specific or too general
- Nine suitable categories are identified (next slide).

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CATEGORIZATION TYPES PREVIOUSLY USED IN RWM

Often used categorizations:

- Parameter, model or scenario uncertainties
- Order of relative magnitude
- Epistemic or aleatory
- Parameter applicability in models (globally to specific)
- Relevance for RWM
- Temporal order of occurrence
- Assignment to system (component or process)
- Type of heterogeneity

Categorization specific to EURAD:

- EURAD Roadmap / Goal Breakdown Structure = Themes, Subthemes & Domains

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UNCERTAINTY CATEGORIZATION - SPECIFIC TO EURAD ...

- EURAD Roadmap / Goal Breakdown Structure (GBS) = Themes, Subthemes & Domains
- In UMAN it was advised already at an early stage to categorize uncertainties according to the occurrence by system phenomena, following the themes and subthemes of the EURAD GBS.
- All EURAD documents follow that approach, i.e. respective information is already pre-categorized (see task 3 subdivisions of UMAN and also the KMS Domain insight documents)

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UNCERTAINTY QUANTIFICATION

Here: Focus on numerical uncertainties => reference to other types (from slide #24 by Tim Hicks):

- programme uncertainties
- uncertainties associated with the **initial characteristics of the waste, site and engineered components**
- uncertainties in the **evolution of the disposal system and its environment** including the effects of events and processes that may affect the initial characteristics of the disposal facility (e.g., hazards that may occur during construction and operation)
- uncertainties associated with the **data** tools and methods **used in the safety case**
- uncertainties associated with the completeness of the FEPs considered in the safety case

The last part of this lecture briefly points to the evaluation as well as quantification of uncertainties:

- Incomplete information (missing data due to too large efforts or lacking access in principle)
- Imprecise data (non-ideal experiments: method, device, boundary conditions etc.)
- Inherent heterogeneities (in space and/or time), i.e. variability

The following lectures by Klaus-Jürgen Röhlrig and Dirk-Alexander Becker will provide more details.

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NUMERICAL EVALUATION

- Three levels of uncertainty evaluation:
 - Experimental stage: Specific designed experiments allow to determine and quantify uncertainties of single uncertainty components.
 - Uncertainty model: Conceptual models are transformed into mathematical models.
 - Mathematical theory: Deterministic/probability statistics, worst case analysis and fuzzy set theory.
- Most reports and experts use **probability statistics**.
Two main branches are frequency statistics and Bayesian statistics
- A frequently used mathematical tool to characterize the value range and the likelihood are probability density functions (pdf).
- **Worst-case analysis**
- **Fuzzy set theory**
- **Challenge:** transfer verbal descriptions into numerical approaches.



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SUMMARY: PROPOSED WORKFLOW FOR NUMERICAL UNCERTAINTY TREATMENT

- Identify all FEPs (features/events/processes)
- Assign importance to the FEPs (related to the safety case) following the GBS
- Assign numerical models to all processes (Top-Down or Bottom-Up)
- Check available parameterization
- Categorize uncertainties to ease further processing
- Assign numerical uncertainties to model parameters, convert information into numerics if necessary and possible, e.g. by applying fuzzy theory
- Derive pdfs
- Define appropriate target function(s) based, e.g., on the regulatory framework
- Apply uncertainty and sensitivity analysis
- Re-iterate on importance and identify **uncertainties that need treatment**
- Start management of uncertainties, e.g. by model reduction or fixation (see previous talk by Tim Hicks)
 - Reduce
 - Mitigate
 - Avoid

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CONCLUSIONS

- “Relevance/Importance” and associated criteria for them with respect to the various topics require a more intensive discussion
- Have a more detailed look on the temporal order of occurrences of uncertainties
- Reduce uncertainties due to poor communication between “applied” persons (field expert, lab expert, etc.) and “geeks” (safety assessor, modeller, etc.).
- Better connect transition from geosphere models to biosphere models
- Investigate the issue of uncertainty correlation
- Extend codes to make directly use of uncertainties and combine uncertainty components in a model other than additive



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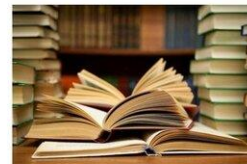
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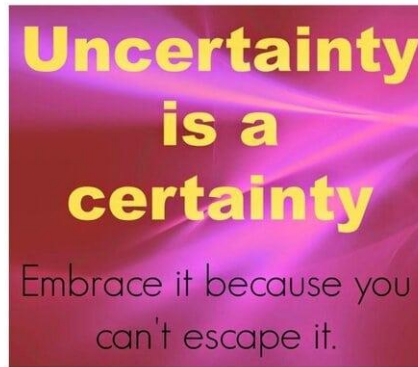
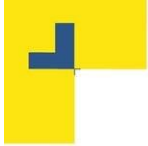
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Thank you for your attention !

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Appendix D. Slides of lecture A.5

A.5 Approaches to uncertainty and sensitivity analysis – Overview

Prepared by Klaus Röhlig, TUC



APPROACHES TO UNCERTAINTY AND SENSITIVITY ANALYSIS – OVERVIEW

EURAD Training on Uncertainty Management, A5
Klaus-Jürgen Röhlig • TU Clausthal, Germany



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Röhlig:

Approaches to uncertainty and sensitivity analysis – Overview

1



KEY WORDS.

- **Uncertainty Analysis**
- **Sensitivity Analysis**
- **Probabilistic Methods**
- **Variance-based Methods**

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Approaches to uncertainty and sensitivity analysis – Overview



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CONTENTS.

- Learning outcomes.
- Introduction.
- Addressing uncertainty quantitatively: Methods.
- Probabilistic methods: underlying assumptions, strategy and workflow.
- Sampling methods.
- Uncertainty analysis (UA): Statistics for calculated indicators and quantities of interest.
- Sensitivity analysis (SA).
 - Local methods.
 - Screening.
 - Global / probabilistic SA.
- Summary and conclusions.
- Bibliography / References.

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LEARNING OUTCOMES.

After the completion of this training unit/lesson, the students/participants should be able to....

- “List the approaches available to perform uncertainty and sensitivity analyses and discuss their pros and cons” (from syllabus), which includes ...
 - Understanding the underlying assumption of the methods and thus their limitations
 - Knowing, in principle, which SA methods provide which kind of information
 - Knowing where to look for further detail ☺

➤ cf. Tim’s lecture

ELEMENTS OF AN UNCERTAINTY MANAGEMENT STRATEGY

- An uncertainty management strategy generally involves the following steps
 - identification and characterisation of uncertainties to be managed
 - analysis of the safety relevance of uncertainties
 - representation of safety-relevant uncertainties in the safety assessment
 - identification of uncertainties that need to be reduced, mitigated or avoided based on assessment results
 - actions to avoid, mitigate or reduce uncertainties
- Views developed as part of UMAN Subtask 2.1, including questionnaire on uncertainty management sent to UMAN participants and literature review

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Date





INTRODUCTION.

- **So far, you have learned about ...**
 - Uncertainty types
 - Uncertainty management approaches in a programme
- **To inform such approaches, it is helpful if uncertainties and their relevance to safety can be quantified**
- **Vinzenz gave you an overview about quantifying the uncertainties themselves, but what about their **impact on / relevance to safety?****
- **For this, we need to propagate them through **model simulations****
- **Unfortunately, quantification is not always possible / appropriate ...**
- **The „usual“ approach in safety assessment includes ...**
 - Develop **scenarios**, i. e. broad-brush descriptions of potential system evolutions – a means to address uncertainty about the future
 - **Computer simulations** addressing system components and processes (process models) or the whole system (system-level models)

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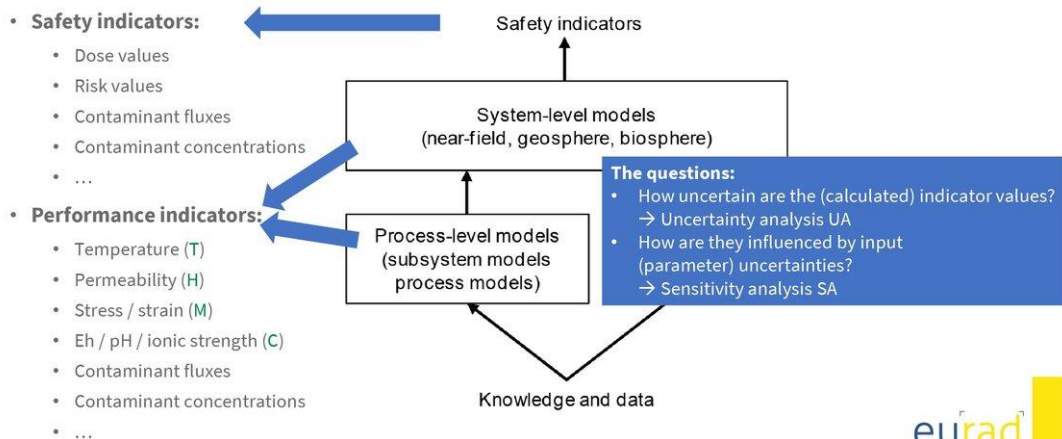
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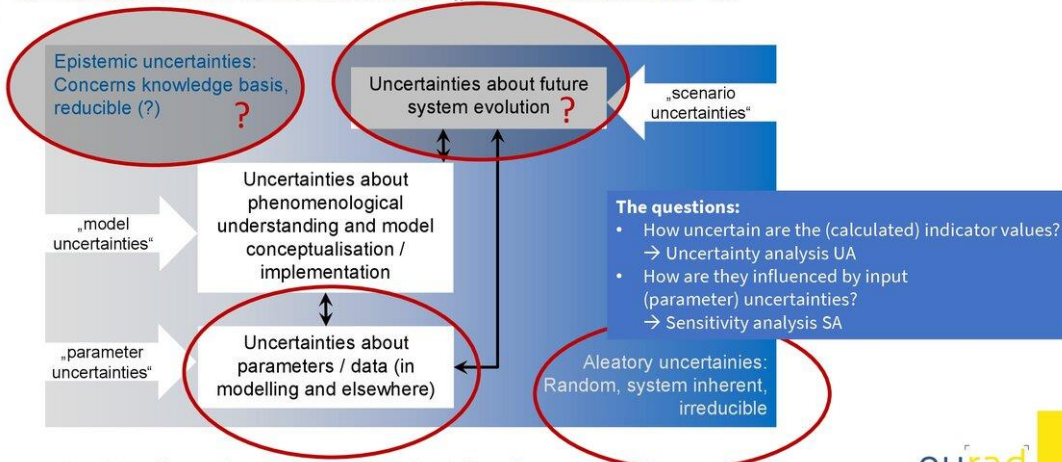
INDICATORS, PROCESS AND SYSTEM-LEVEL MODELS

(SOURCE: MESA REPORT, OECD/NEA)





INTRODUCTION. TALKING ONLY ABOUT „KNOWN UNKNOWN“ ...



Based on <http://www.ip-pamina.eu/downloads/pamina.m1.2.1.pdf>

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ADDRESSING UNCERTAINTY QUANTITATIVELY: METHODS.

- Let's assume we did everything reasonably conceivable to qualify the scenarios and models we are working with (good science, structured scenario approaches, model qualification, ...) and are now aiming just at these two statements:
 - How uncertain are the (calculated) indicator values? → Uncertainty analysis UA
 - How are they influenced by input (parameter) uncertainties? → Sensitivity analysis SA
- What can we do?
 - Testing the model using various input parameter sets ("realisations").
 - But this can be done in various ways, we need to be clever!
 - We might have limitations concerning the number of realizations / model runs.
 - Given such boundary conditions, we want to "squeeze out" as much information as possible.
 - By doing so, our aims might be:
 - Quantification of uncertainty
 - Traceability, reproducibility
 - Transparency, good communication
 - Need for clever design of computer experiments

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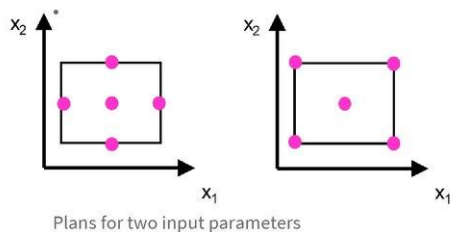
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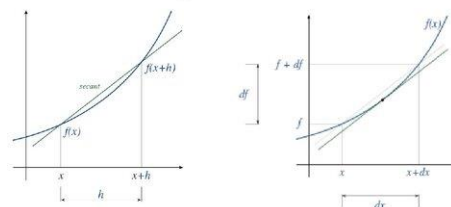


ADDRESSING UNCERTAINTY QUANTITATIVELY: METHODS.

- **Deterministic methods: Testing one-by-one according to a plan, e.g. screening by using ...**
 - Minimum – best estimate (guess) – maximum
 - [favorable] – best estimate (guess) – unfavorable (“conservative”) (conservatism needs to be demonstrated!)
 - phenomenological – conservative – penalizing / alternative (from Andra Dossier 2005)



- **Local methods – gradient-based information ...**



Estimating sensitivities using numerical derivatives (difference quotients) (picture source: Wikipedia)



ADDRESSING UNCERTAINTY QUANTITATIVELY: METHODS.

- **Deterministic methods: Testing one-by-one according to a plan, e.g.**
 - Minimum – best estimate (guess) – maximum
 - [favorable] – best estimate (guess) – unfavorable (“conservative”) (conservatism needs to be demonstrated!)
 - phenomenological – conservative – penalizing / alternative (from Andra Dossier 2005)
- **Probabilistic methods,**
 - i. e. methods utilizing the theory and toolboxes of probability theory and statistics**
 - Require two types of models:
 - One describing physico-chemical (THMC) phenomena, the other one describing input uncertainty
 - The latter one: Need to derive a [joint pdf](#) for the input parameter set (cf. Vinzenz’ lecture)





ADDRESSING UNCERTAINTY QUANTITATIVELY: METHODS.

- **Deterministic methods: Testing one-by-one according to a plan, e.g.**
 - Minimum – best estimate (guess) – maximum
 - [favorable] – best estimate (guess) – unfavorable (“conservative”) (conservativity needs to be demonstrated!)
 - phenomenological – conservative – penalizing / alternative (from Andra Dossier 2005)
- **Probabilistic methods,
i. e. methods utilizing the theory and toolboxes of probability theory and statistics**
 - Sometimes questioned as being inappropriate for epistemic uncertainties, this is often linked to other proposals such as ...
 - Interval math
 - Fuzzy methods (Zadeh, can work with non-additive measures)
 - Possibility theory (Zadeh, working with “possibility” & “necessity”)
 - Dempster-Shafer theory of belief (or evidence theory, “belief” & “plausibility”)
 - “upper” & “lower” probabilities (Dempster)
 - Other hybrid methods

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ADDRESSING UNCERTAINTY QUANTITATIVELY: METHODS.

- **Deterministic methods: Testing one-by-one according to a plan, e.g.**
 - Minimum – best estimate (guess) – maximum
 - [favorable] – best estimate (guess) – unfavorable (“conservative”) (conservatism needs to be demonstrated!)
 - phenomenological – conservative – penalizing / alternative (from Andra Dossier 2005)
- **Probabilistic methods,
i. e. methods utilizing the theory and toolboxes of probability theory and statistics**
 - Sometimes questioned in literature as being inappropriate for epistemic uncertainties ...
- **... but nevertheless widely used**
 - based on the arguments that Kolmogorov’s axioms do not require a frequentistic interpretation of probability,
 - the applicability of Bayesian information updating and
 - “justified” by the power of the toolboxes,

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PROBABILISTIC METHODS: UNDERLYING ASSUMPTIONS, STRATEGY AND WORKFLOW.

The steps.

- Quantifying uncertainties via pdfs. Watch out for dependencies!!!
- Sampling
- Model runs for each realization
- UA, possibly for “quantities of interest”
- SA

The underlying assumptions.

- The „true“ parameter set (x_1, x_2, \dots, x_n) is a realisation of an n-dimensional random variable (X_1, X_2, \dots, X_n) with a joint pdf P
- The sampled parameter sets (realisations) $\{(x_1^i, x_2^i, \dots, x_n^i), i=1, \dots, m\}$ are a good approximation of the „true“ pdf P
- For each realisation No. i, the (deterministic!) THMC model produces the „correct“ result $f = f(x_1^i, x_2^i, \dots, x_n^i)$
- The $\{f, i=1, \dots, m\}$ are a good approximation of the output distribution
- The joint set of inputs and outputs $\{(x_1^i, x_2^i, \dots, x_n^i, f), i=1, \dots, m\}$ is a good approximation of the „true“ relationships

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SAMPLING METHODS: A WISH LIST.

We would like to get a series of parameter sets which ...

- ... approximates the given joint pdf P
- ... the realisations of which are independent
- ... which leaves no „gaps“ in the parameter space
- ... the sample size of which is not greater than ???

As we will see ...

- Some of the wishes compete with each other.
- Different sampling schemes satisfy different wishes differently well.

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Approaches to uncertainty and sensitivity analysis – Overview

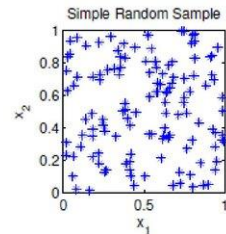
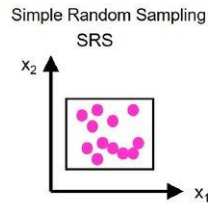




SAMPLING METHODS: A WISH LIST.

We would like to get a series of parameter sets which ...

- + ... approximates the given joint pdf P
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Röhlig:

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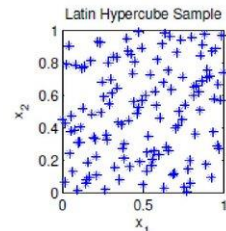
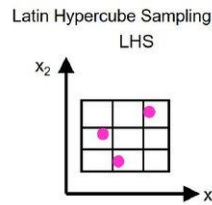
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SAMPLING METHODS: A WISH LIST.

We would like to get a series of parameter sets which ...

- + ... approximates the given joint pdf P (choose intervals of equal probability!)
- ... the realisations of which are independent
- ± ... which leaves no „gaps“ in the parameter space
- ± ... the sample size of which is not greater than ???



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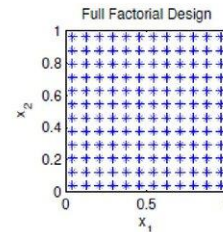
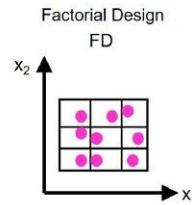
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SAMPLING METHODS: A WISH LIST.

We would like to get a series of parameter sets which ...

- + ... approximates the given joint pdf P (choose intervals of equal probability!)
- ... the realisations of which are independent
- + ... which leaves no „gaps“ in the parameter space
- ... the sample size of which is not greater than ??? (way out: Fractional FD)



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Röhlig:

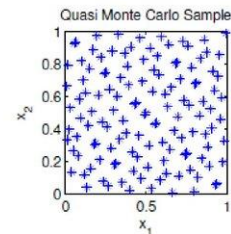
Approaches to uncertainty and sensitivity analysis – Overview



SAMPLING METHODS: A WISH LIST.

We would like to get a series of parameter sets which ...

- + ... approximates the given joint pdf P
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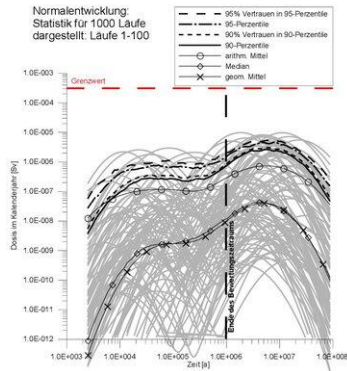
Röhlig:

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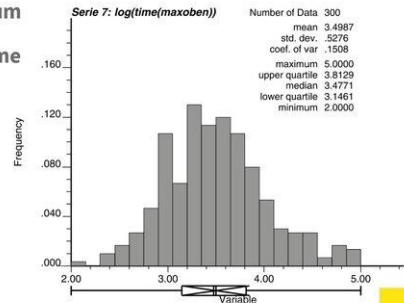
UNCERTAINTY ANALYSIS (UA): STATISTICS FOR CALCULATED INDICATORS AND QUANTITIES OF INTEREST.

Statistics for time series ...



... or for „quantities of interest“ such as:

- Maximum
- Time of maximum
- Integral over time



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Röhlig:

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UNCERTAINTY ANALYSIS (UA): STATISTICS FOR CALCULATED INDICATORS AND QUANTITIES OF INTEREST.

Note:

- In principle, all „classic“ statistical estimates (for moments, quantiles) could be applied.
- The statistical confidence in the estimate depends on the sample size, though.
- Many confidence statements presuppose a certain probability type (e.g. a normal distribution). There are, however, also possibilities of avoiding such presumptions.
- If the quantity of interest is an indicator addressed in regulations (e. g. by setting a limit for individual effective annual dose), you need to make up your mind about what to do if single (perhaps very few, perhaps just one) realization(s) exceed the limit!
- In the case of risk-based regulations, one might use probabilistics to estimate it (since risk in the conventional sense „consequence times probability“ is an expectation of the consequence)
– but there are other ways as well!

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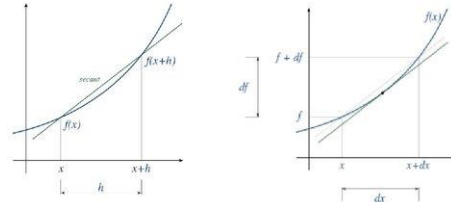
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SENSITIVITY ANALYSIS (SA): LOCAL METHODS.

- Study model (deterministically) around a default or reference value / base case / working point
- Partial derivatives or difference quotients varying **one** input parameter **at a time** (OAT)
- However: Comparability of parameters? (different units)
- Suggestion: Importance measure

$$DIM_i = \frac{\partial f(x^0)}{\partial x_i} dx_i \left(\sum_{i=1}^d \frac{\partial f(x^0)}{\partial x_j} dx_j \right)^{-1}$$



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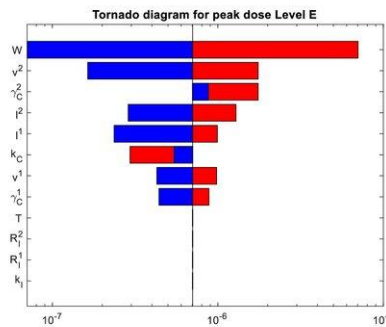
Röhlig:

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SENSITIVITY ANALYSIS (SA): SCREENING

- Purpose: Identify uninfluential input parameters with relative few model evaluations
- Tornado diagrams: OAT variations from base case (“worst” and “best” cases)
→ calculate output deviations



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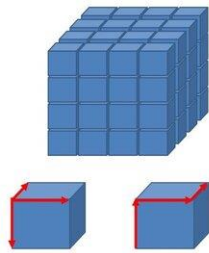
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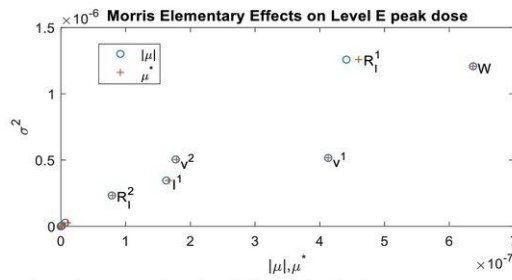
SENSITIVITY ANALYSIS (SA): SCREENING

- Purpose: Identify uninfluential input parameters with relative few model evaluations
- Tornado diagrams: OAT variations from base case
→ calculate output deviations
- Morris methods: Randomized choice of hypercube, OAT designs, take e.g. mean and variance of the (absolute) difference quotients as sensitivity measures



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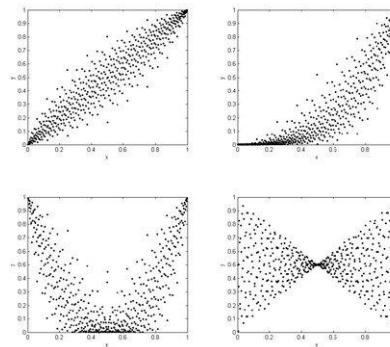
Röhlig:



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GLOBAL SA: WHICH EFFECTS CAN BE DETECTED?



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Röhlig:

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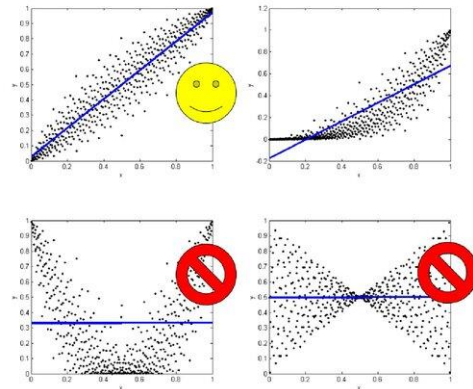
GLOBAL SA: CORRELATION ANALYSIS / PEARSON COEFFICIENTS / LINEAR REGRESSION

- Estimate e.g. correlation coefficient

$$\rho(X_i, Y) = \frac{\text{cov}(X_i, Y)}{\sqrt{v(X_i)v(Y)}}$$

by

$$CC = \frac{\sum_{i=1}^n (x_{ij} - \bar{x}_j)(y_i - \bar{y})}{\left[\sum_{i=1}^n (x_{ij} - \bar{x}_j)^2\right]^{1/2} \left[\sum_{i=1}^n (y_i - \bar{y})^2\right]^{1/2}}$$



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Röhlig:

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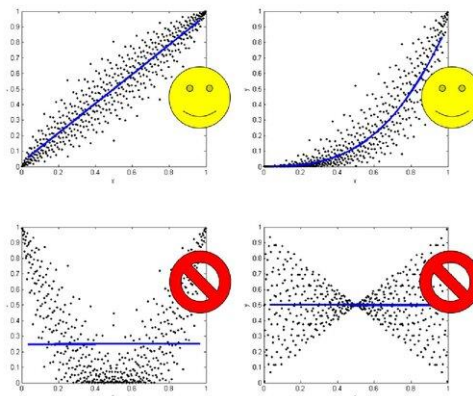
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GLOBAL SA: RANK CORRELATION ANALYSIS / SPEARMAN COEFFICIENTS

- Replace parameters and outputs by their ranks, i.e. order them according to value and assign numbers in that order („ranks“)
- Then calculate correlation coefficients of these ranks



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GLOBAL SA: VARIANCE-BASED METHODS

- **Idea: Decomposition**

$$Y = f(X_1, \dots, X_k)$$

$$= f_0 + \sum_i f_i(X_i) + \sum_{i < j} f_{ij}(X_i, X_j) + \dots + f_{1\dots k}(X_1, \dots, X_k)$$

- **For independent inputs:**

$$V = \text{Var}(Y) = \sum_i V_i + \sum_{i < j} V_{ij} + \dots + V_{1\dots k}$$

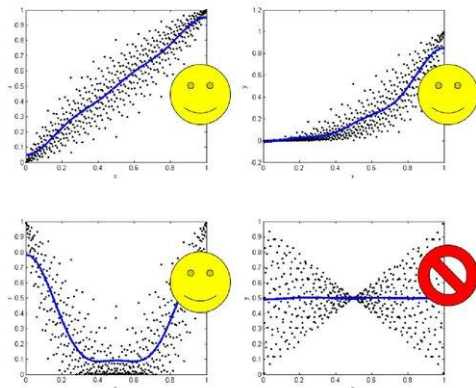
with

$$V_i = \text{Var}(E[Y | X_i])$$

$$V_{ij} = \text{Var}(E[Y | X_i, X_j]) - V_i - V_j$$

$$V_{ijk} = \text{Var}(E[Y | X_i, X_j, X_k]) - V_{ij} - V_{jk} - V_{ki} - V_i - V_j - V_k$$

...
 → Sobol's indices $S_i = \frac{V_i}{V}$, $S_{ij} = \frac{V_{ij}}{V}$ etc.



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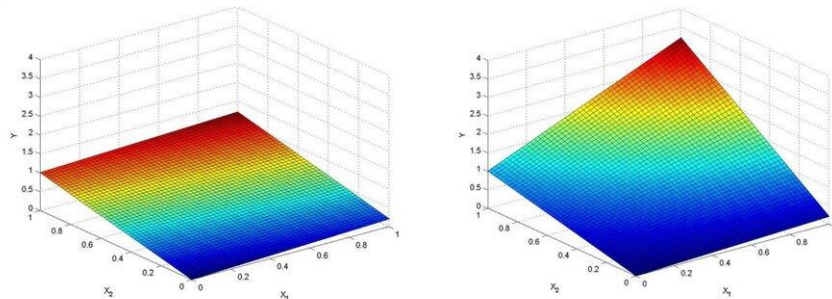
Röhlig:

Approaches to uncertainty and sensitivity analysis – Overview



GLOBAL SA: VARIANCE-BASED METHODS ...

- ... are able to detect non-linear and even non-monotonic effects
- ... and interactions



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Röhlig:

Approaches to uncertainty and sensitivity analysis – Overview



GLOBAL SA: VARIANCE-BASED METHODS ...

- ... are able to detect non-linear and even non-monotonic effects
- ... and interactions
- ... but indices, especially higher ones, are not easy to estimate,
- ... methods often require specific space-filling sampling schemes and high sample sizes

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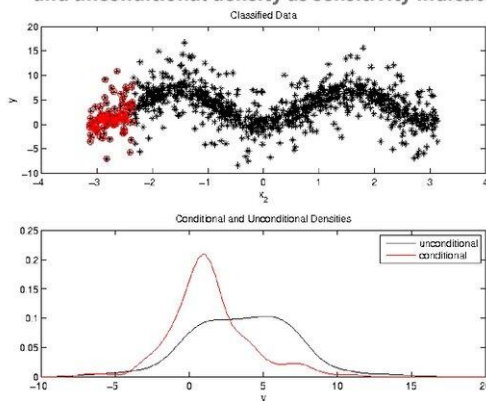
Röhlig:

Approaches to uncertainty and sensitivity analysis – Overview



GLOBAL SA: DENSITY-BASED METHODS

- No regression model, distance between conditional and unconditional density as sensitivity indicator



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Röhlig:

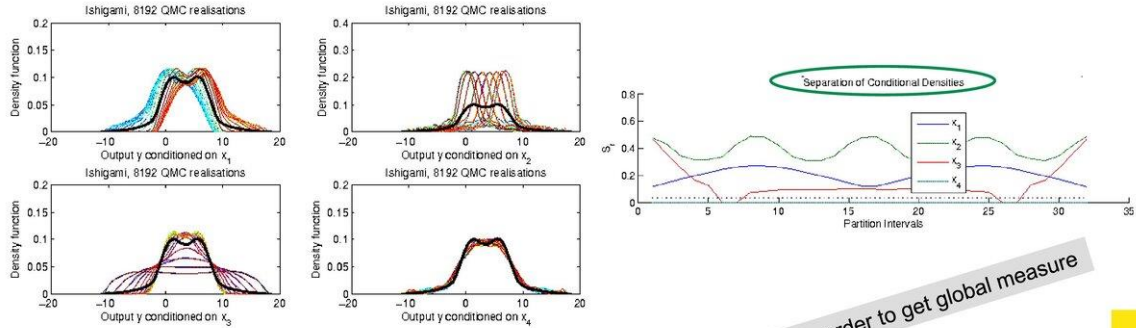
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GLOBAL SA: DENSITY-BASED METHODS – SEVERAL INPUTS

- No regression model, distance between conditional and unconditional density as sensitivity indicator



Then: Average in order to get global measure

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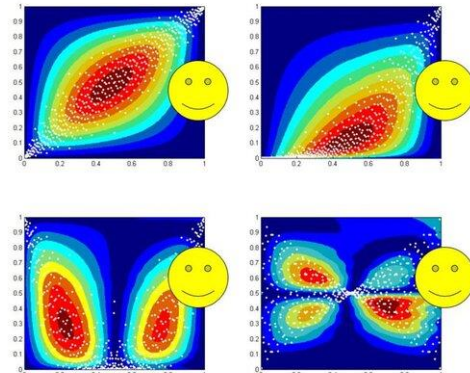
Röhlig:

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GLOBAL SA: DENSITY-BASED METHODS – SEVERAL INPUTS

- No regression model, distance between conditional and unconditional density as sensitivity indicator



By the way: The synthetic example shown on the two previous slides (the "Ishigami case") is different from the one shown on slides 24-26 & 31 (i. e. here)!

- "Ishigami":
Parameter #4 is a dummy with no impact on the result
- This slide: Parameter 4 has an influence which cannot be detected by moment-based methods

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SA: VARIOUS METHODS WITH PROS AND CONS, OFTEN COMPLICATED AND EXPENSIVE

Advice

- Start with graphics (not addressed in this lecture, sorry ☹)
- Then step by step from simple to complicated
- For each step: check to which degree your results explain the behavior (goodness of fit, sum of indices)
- Stop if explanation is „satisfactory“

→ Feedback to decision makers!

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SUMMARY AND CONCLUSIONS.

- This lecture was about quantifying model output uncertainties (UA) and their relationship to input uncertainties (SA)
- A combination of deterministic and probabilistic methods is the established (and powerful) way of doing this
- Especially SA is tricky:
 - Methods often expensive, sometimes hard to understand
 - Ongoing research about advanced methods
 - Gap between practitioners on one hand and SA researchers on the other
 - Note graphical methods (not addressed in this lecture)!
 - Graphics are often instructive
 - However, rationale behind is not always easy to understand

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Röhlig:

Approaches to uncertainty and sensitivity analysis – Overview





BIBLIOGRAPHY / REFERENCES.



<https://doi.org/10.2172/1822591>

... in preparation

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Approaches to uncertainty and sensitivity analysis – Overview



Appendix E. Slides of lecture A.6

A.6 Approaches to uncertainty and sensitivity analysis – Practical examples from the DONUT project

Prepared by Dirk Alexander Becker, GRS



APPROACHES TO UNCERTAINTY AND SENSITIVITY ANALYSIS Practical application

Dirk-Alexander Becker • WP UMAN / DONUT



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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KEY WORDS.

- Geological disposal
- Performance assessment
- Numerical simulation
- Uncertainty analysis
- Sensitivity analysis

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CONTENTS

- **Learning outcomes**
- **Introduction**
- **Methods of global sensitivity analysis:**
 - Overview, methods and problems
- **Practical examples**
 - GRS LILW model
 - SNL crystalline reference case
 - UDC Reactive transport model
 - Overview of DONUT investigations
- **Summary and conclusions**
- **Bibliography and/or References**

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LEARNING OUTCOMES

After the completion of this training unit/lesson, the students/participants should be able to

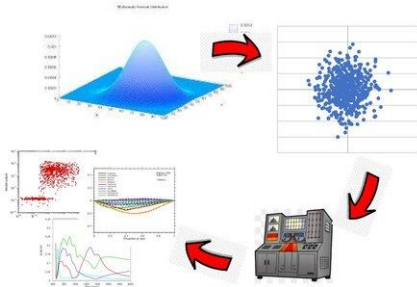
- understand why and how probabilistic uncertainty and sensitivity analysis is applied
- understand the outcome of probabilistic uncertainty and sensitivity analysis
- distinguish different types of sensitivity analysis methods and understand their particularities in practice

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INTRODUCTION



- **Why do we perform uncertainty and sensitivity analysis?**
 - Assess the overall uncertainty of a model calculation,
 - Identify influential parameters,
 - Improve model understanding.
- **You have learned in the previous talk ...**
 - ... what the general idea of probabilistic UA/SA is,
 - ... which types of mathematical method exist,
 - ... how one can interpret the results.
- **In this talk you will ...**
 - ... get acquainted with some UA/SA methods,
 - ... see a few actual UA/SA studies,
 - ... learn which practical problems might occur,
 - ... learn what one can conclude from the results.

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METHODS OF GLOBAL SENSITIVITY ANALYSIS – OVERVIEW (NOT COMPLETE!)

Uncertainty

- Scatterplots
- Horsetail plots
- Histograms
- Output distribution characteristics

Linearisation

	Direct	Rank-based
Regression	SRC	SRRC
Correlation	Pearson	Spearman
Partial correlation	PCC	PRCC

Moment-independent / Nonparametrical / density-based

- Two-sample tests
 - Kolmogorov-Smirnov
 - Cramer-von-Mises
- CDF distance
 - Gini
 - Borgonovo
 - Pianosi-Wagener (PAWN)

Screening

- Morris
- Tornado diagram

Variance-based

	Tailored sampling	Free sampling
1 st order	SOBOL FAST EFAST RBD	EASI COSI
Higher order	SOBOL	Metamodeling
Total order	SOBOL EFAST	PCE RS-HDMR

Other

- Shapely
- VARS

Graphical

- Scatterplots
- CSM
- CSV
- CUSUNORO

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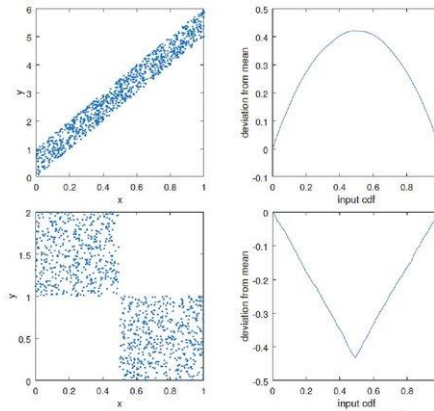




IN SHORT: CUSUNORO

Cumulative Sum of Normalised Reordered Output (Plischke 2012)

- **Analysis of a set of model runs**
 - Reorder the model runs
 - increasing value of the parameter of interest.
 - For each value y_i calculate its difference to the mean and normalise by the standard deviation
 - $c_i = \frac{y_i - \bar{y}}{\sigma_y}$.
 - Calculate the cumulative sum
 - $s_i = \sum_{l \leq i} c_l$.
 - Plot s_i/n over i/n . (sometimes: $-s_i/n$)
- **Deviation of the curve from the horizontal line indicates parameter sensitivity**
 - the more curved, the more sensitive!



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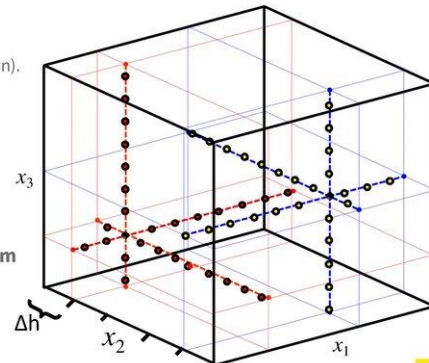
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IN SHORT: VARS

Variogram analysis of response surfaces (Razavi and Gupta, 2016)

- **Parameters x_i are normalized into [0, 1] intervals**
 - Each axis is discretized with constant parameter increments Δh (resolution).
- **Perform quasi Monte-Carlo sampling defining first the so-called “star centers”**
 - Latin hypercube sampling (LHS), Halton, ...
- **Draw lines parallel to the axes & evaluate the function Z at all the discretized points.**
- **Estimate the experimental variogram of Z along each direction from the values of $\{Z_i, x_i\}$.**
- VARS includes Sobol and Morris analysis as specific cases.
- (see <https://www.youtube.com/watch?v=YxeMdA8QIfU>)



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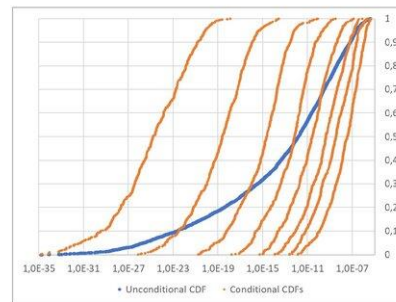
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IN SHORT: PAWN

Pianosi and Wagener (2015, 2018)

- Calculate the unconditional cumulated density function (CDF).
- Divide the model runs into a number of groups
 - according to values of the parameter of interest,
 - equal group population highly recommended (though not originally proposed by the authors).
- Calculate the conditional CDF for each group.
- Calculate the maximum absolute difference between conditional and unconditional CDF.
- Calculate the mean of all these differences over the groups:
 - this is a measure for the sensitivity of the parameter,
 - always between 0 and 1.
- PAWN is subject to random noise:
 - does not reach 0.



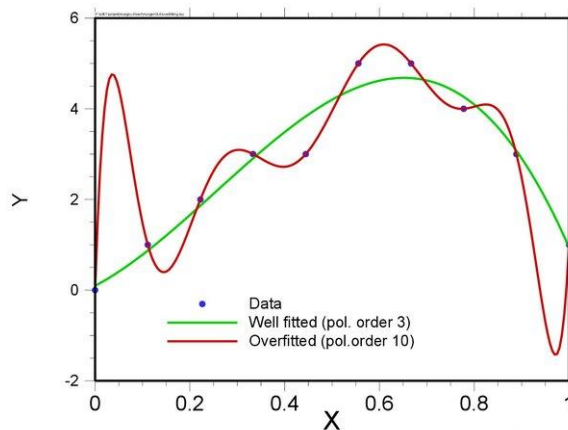
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METAMODELING: THE PROBLEM OF OVERFITTING

- **Polynomial metamodel:**
 - to be fitted to the given data,
 - polynomial order has to be defined,
 - a higher order hits data points more exactly but
 - can lead to fitting artifacts.
- **Chose the polynomial order with care!**
 - Higher is not necessarily better,
 - too high orders can completely corrupt the metamodel.



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PRACTICAL EXAMPLE 1: SALT LILW MODEL (GRS)



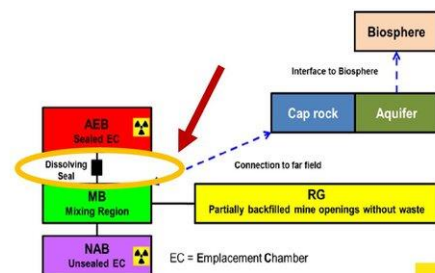
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SALT LILW MODEL: DESCRIPTION

- **Repository for Low- and Intermediate-Level Waste (LILW) in rock salt**
- **Simplified model for radionuclide transport**
 - Near field
 - one sealed emplacement chamber with longer-lived waste
 - one unsealed emplacement chamber with short-lived waste
 - one compartment for all mine openings without waste
 - one mixing compartment
 - Far field
 - 1D-transport through cap rock and aquifer
 - Biosphere
 - annual dose to a human individual



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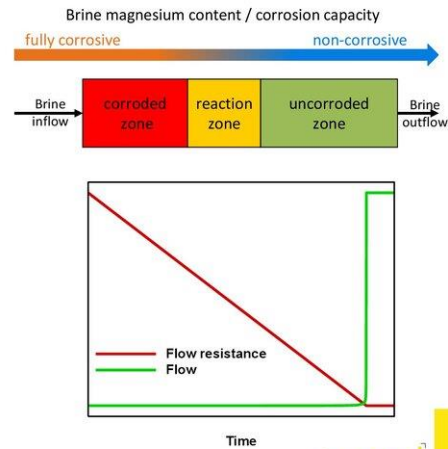
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SALT LILW MODEL: SEAL CORROSION

- The seal of the sealed emplacement chamber is made of salt concrete, which is chemically instable against magnesium-containing brine.
- The corrosion process disintegrates the salt concrete while consuming magnesium, so that the brine loses its corrosion capacity.
- This leads to a narrow reaction zone slowly propagating through the seal, which can be idealised as a sharp front.
- The permeability is assumed to increase by four orders of magnitude at the front.
- Since the flow resistance is dominated by the non-corroded part of the seal, it decreases by several orders of magnitude nearly suddenly when the front reaches the end of the salt concrete body.



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SALT LILW MODEL: PLANNING OF PROBABILISTIC ANALYSIS

- **Large number of model parameters**
 - Which of them are most interesting?
 - 20 parameters were selected for probabilistic variation
 - “Plausible” probabilistic distributions were assigned to all of them
 - **Caution:** For real cases, this should be done with care. The pdfs should reflect the “real” uncertainty as well as possible to produce reliable UA/SA results. This is an extra task and requires a lot of effort!
- **A rough sensitivity analysis lead to some findings:**
 - Six parameters seem to be most influential.
 - Five parameters are less sensitive but seem to play a certain role.
 - Nine parameters show negligible sensitivity.
- **Further analysis with different sets of parameters**
- **Different sampling strategies**
 - random sampling
 - quasirandom sampling (LpTau)
 - sample sizes from 512 to 16384

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SALT LILW MODEL: PARAMETERS

Parameter	Type of pdf	Range of pdf parameters	Standard value	Parameter	Type of pdf	Range of pdf parameters	Standard value
GasEntryP:				RGGasProd:			
Gas entry pressure	Uniform	0 - 2.5	2.0	Proportion of the material involved in gas production in RG	Triangular	0.1 - 0.8 - 1.0	0.8
InitialPermSeal:				NABConv:			
Initial permeability of dissolving seal	Log-normal	$\mu=41.0605$ $\sigma=1.9809$	1.0E-18	Factor of local convergence variation in NAB	Log-uniform	0.05 - 5.0	0.2
RefConv:				MBConv:			
Reference convergence rate	Log-uniform	1.0E-5 - 1.0E-4	4.0E-5	Factor of local convergence variation in MB	Log-uniform	0.075 - 0.75	0.2
AEBConv:				DiffCoeff:			
Factor of local convergence variation in AEB	Log-uniform	0.05 - 5.0	1.0	Diffusion coefficient	Log-uniform	1.0E-10 - 1.0E-8	1.0E-9
GasCorrPE:				RefPor:			
Organics corrosion rate	Log-normal	$\mu=12.6642$ $\sigma=1.1177$	1.0E-5	Reference porosity	Triangular	0.15 - 0.3 - 0.4	0.3
TBrine:				FacDisp:			
Time of brine intrusion	Log-normal	$\mu=8.8857$ $\sigma=0.6933$	7500	Longitudinal dispersion length	Triangular	0.5 - 1.0 - 2.0	0
BrineMgSat:				ConvFak:			
Relative magnesium saturation of brine	Triangular	0 - 0.1 - 1.0	0.1	Variation factor for sheeting	Uniform	0 - 2.0	1.0
RCCConv:				PorDebris:			
Factor of local convergence variation in RG	Log-uniform	0.25 - 2.5	1.0	Porosity of debris from sheeting	Uniform	0.25 - 0.5	0.4
GasCorrFe:				C14Inv:			
Metal corrosion rate	Log-normal	$\mu=6.6728$ $\sigma=1.1177$	4.0E-3	Variation factor for C-14 inventory	Uniform	0 - 2.5	1.0
AEBGasProd:							
Proportion of the material involved in gas production in AEB	Triangular	0.1 - 0.8 - 1.0	0.8				
NABGasProd:							
Proportion of the material involved in gas production in NAB	Triangular	0.1 - 0.8 - 1.0	0.8				

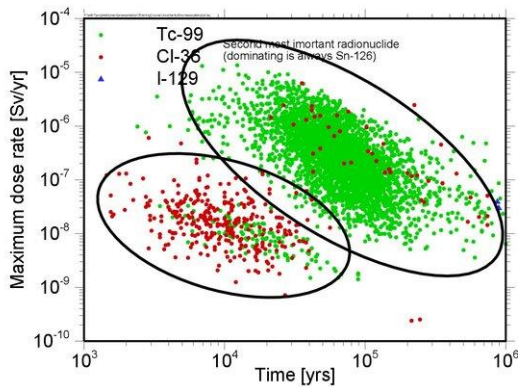
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SALT LILW MODEL (20 PARAMETERS): UNCERTAINTY ANALYSIS

4096 runs



Maximum scatterplot:

- For each individual model run the absolute maximum over time is plotted against its time of occurrence.
- An additional piece of information can be coded via dot colour. Here: the second most important radionuclide (as the most important one is always Sn-126).
- **One can see**
 - The total span of maxima is between $2 \cdot 10^{-10}$ Sv/yr and $2 \cdot 10^{-5}$ Sv/yr
 - Two clouds of dots:
 - lower and earlier maxima (seal has not failed by the end of scenario),
 - higher and later maxima (occur after seal has failed).
 - Cl-36 is important for most runs in the lower cloud, while Tc-99 dominates the upper cloud.

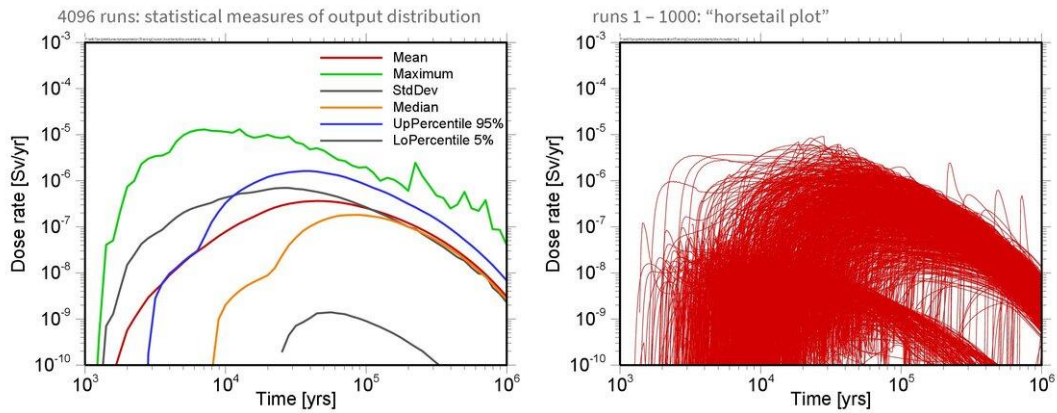
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SALT LILW MODEL (20 PARAMETERS): UNCERTAINTY ANALYSIS



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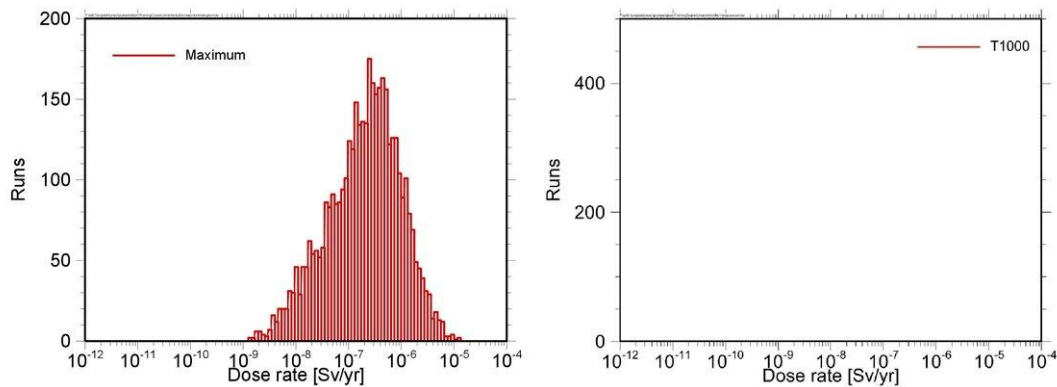
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SALT LILW MODEL (20 PARAMETERS): UNCERTAINTY ANALYSIS



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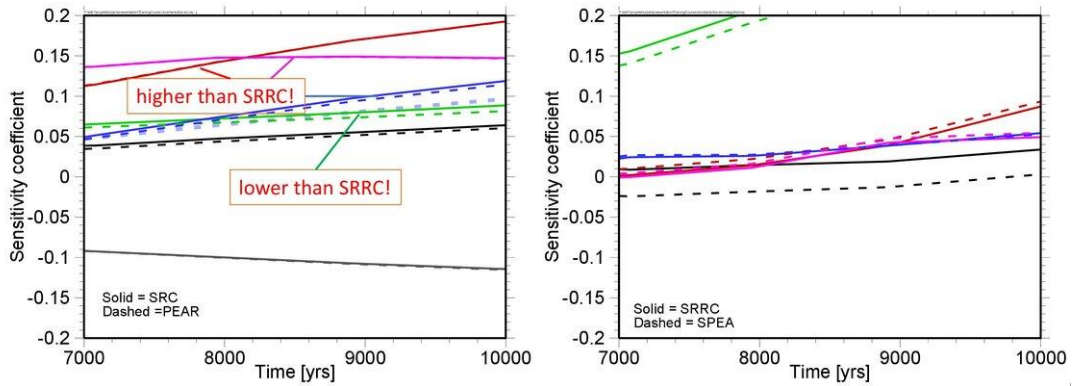
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SALT LILW MODEL (6 PARAMETERS): SENSITIVITY ANALYSIS



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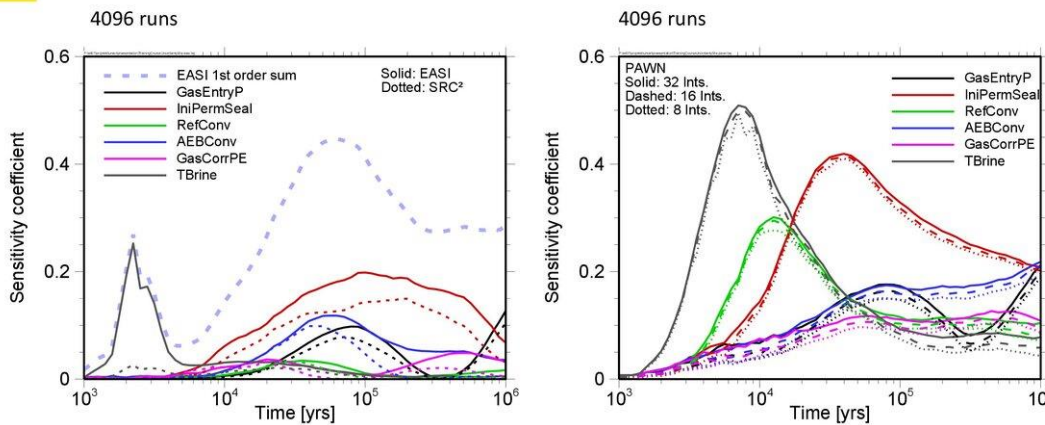
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SALT LILW MODEL (6 PARAMETERS): SENSITIVITY ANALYSIS



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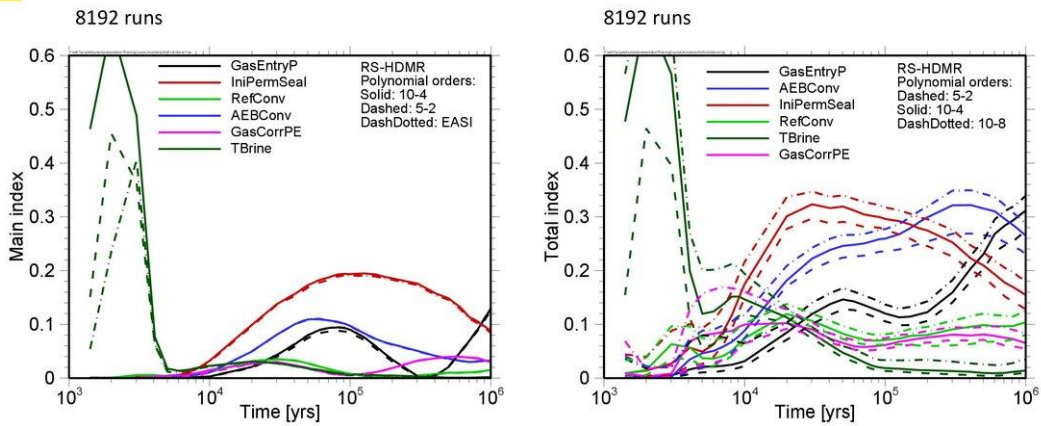
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SALT LILW MODEL (6 PARAMETERS): METAMODEL ANALYSIS



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SALT LILW MODEL (6 PARAMETERS): SENSITIVITY ANALYSIS OUTCOME

- **Despite its nonlinearity the model can be analysed with linearising methods:**
 - mostly fair agreement between SRC and EASI,
 - except: AEBConv at > 200000 years
 - positive and negative linear influences compensate but
 - variance does not become 0.
- **Differences between SRC and SRRC:**
 - SRC > SRRC indicates that the parameter acts mainly on the highest values,
 - SRC < SRRC indicates that the parameter acts mainly on the lowest values.
- **PAWN yields (in part) considerably different results than EASI:**
 - more pronounced dominance of IniPermSeal,
 - much higher sensitivity of TBrine and RefConv,
 - this is a hint to sensitivities that are not well captured by regression or variance:
 - RefConv acts on low values.
- **Metamodeling (RS-HDMR) results are**
 - robust against choice of polynomial orders and well in line with EASI for 1st order,
 - less robust for higher and total order.

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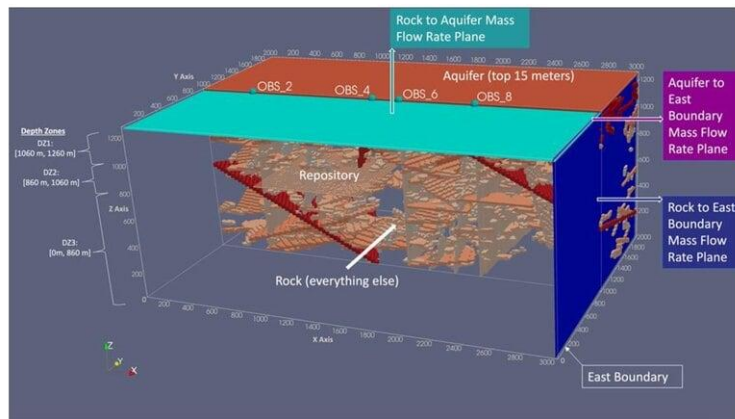
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PRACTICAL EXAMPLE 2: CRYSTALLINE REFERENCE CASE (SNL)



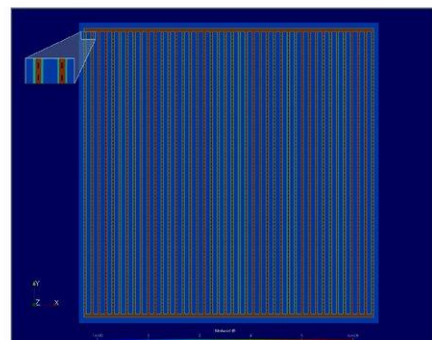
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CRYSTALLINE REFERENCE CASE: MODEL DESCRIPTION

- **Generic repository for Spent Nuclear Fuel (SNF) in fractured crystalline rock**
- **The model describes**
 - Radionuclide release from waste package
 - Migration through buffer material
 - Transport through damaged rock zone (DRZ)
 - Transport through connected fractures to the biosphere
- **Effects**
 - Advective / diffusive flow
 - Diffusion into and out of pores and fractures
 - Sorption on colloids and immobile mineral surfaces
 - Chemical reactions
 - Dissolution and precipitation
 - Radioactive decay and ingrowth
- **Quantity of interest: peak I-129 concentration in aquifer.**



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CRYSTALLINE REFERENCE CASE: DISCRETE FRACTURE NETWORK (DFN)

- The crystalline host rock is disturbed by interconnected fractures.
- The network of discrete fractures serves as a transport path for radionuclides.
- The exact positions and dimensions of the fractures are unknown.
- The fracture network as a whole is characterised by parameters (“graph metrics”):
 - Average number of intersections per fracture (network connectivity),
 - Number of fractures intersecting the repository (number of potential flow pathways from the repository),
 - Shortest travel time between repository and aquifer (ease of flow from repository to aquifer).
- The DFN has an aleatory influence on the model results.

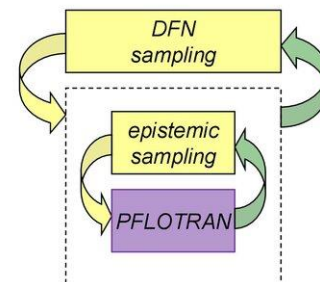
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CRYSTALLINE REFERENCE CASE: PLANNING OF PROBABILISTIC ANALYSIS

- 25 DFN were generated using a specific software:
 - random fractures,
 - influenced by 3 parameters,
 - numbered 1 – 25:
 - no continuous relation between DFN number and properties,
 - fixed order of waste package breach for each DFN.
- For each DFN, 40 sets of epistemic parameters were drawn.
- A total of 1000 model runs were performed.
- Possible evaluations:
 - Total set without graph metrics (1000 runs),
 - Total set with graph metrics (1000 runs),
 - 25 subsets, one for each DFN (40 runs).



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CRYSTALLINE REFERENCE CASE: PARAMETERS

Epistemic Variables				
Input	Description	Unit	PDF type	Range
rateUNF	Fractional dissolution rate of spent (used) nuclear fuel	1/yr	log unif.	$10^{-8} - 10^{-6}$
kGlacial	Glacial till permeability	m ²	log unif.	$10^{-15} - 10^{-13}$
pBuffer	Buffer porosity	-	uniform	0.3 – 0.5
permDRZ	DRZ permeability	m ²	log unif.	$10^{-19} - 10^{-16}$
permBuffer	Buffer permeability	m ²	log unif.	$10^{-20} - 10^{-17}$
meanWPrate	Mean of the truncated log normal distribution on base normalized general corrosion rate (R)	log (1/yr)	uniform	-5.5 – (-4.5)
stdWPrate	Standard deviation of the truncated log normal distribution	log (1/yr)	uniform	0.15 – 0.4
IRF	Instant release fraction	-	uniform	0.038 – 0.156

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CRYSTALLINE REFERENCE CASE: GRAPH METRIC PARAMETERS

Measures of Spatial Heterogeneity				
Graph Metric	Description	Unit	PDF type	Range
STT	The log ₁₀ -transformed relative shortest travel time between repository and aquifer. Relative shortest travel time computed by scaling shortest travel time for each DFN by the median. A measure of ease of flow between repository and aquifer.	log (yr)	uniform	-2.0 – 2.0
aveDegree	Average number of intersections per fracture. A measure of how connected the network is over the entire domain.	-	uniform	3.4 – 3.6
Intersections	Number of fractures intersecting the repository. A measure of number of potential flow pathways out of the repository region.	-	uniform	63 - 101

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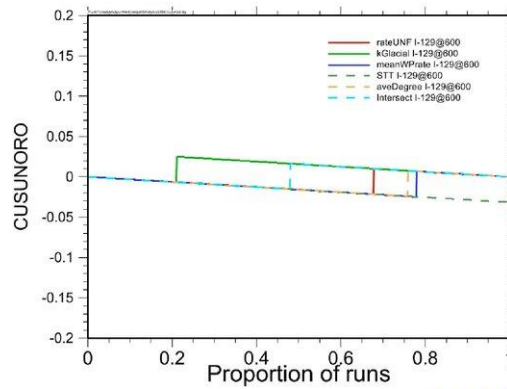
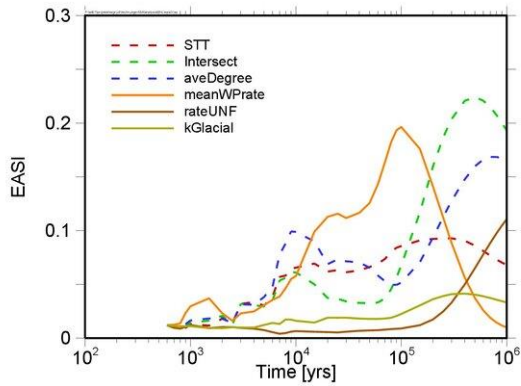
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CRYSTALLINE REFERENCE CASE: TIME-DEPENDENT ANALYSIS



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CRYSTALLINE REFERENCE CASE: TIME-DEPENDENT ANALYSIS

EASI

- **meanWPrate**
 - increasing sensitivity for 100000 years,
 - decreasing sensitivity in the later phase
 - dominant during the first 400000 years.
- **rateUNF and kGlacial**
 - start to become sensitive only after 100000 years,
 - **rateUNF more dominant in the late phase.**
- **graph metric parameters**
 - **clear and high influence in the late phase.**

CUSUNORO

- **meanWPrate**
 - increasing sensitivity for 100000 years,
 - decreasing sensitivity in the later phase
 - dominant during the first 400000 years.
- **rateUNF and kGlacial**
 - start to become sensitive only after 100000 years,
 - **comparably dominant in the late phase.**
- **graph metric parameters**
 - **unclear influence at all times.**

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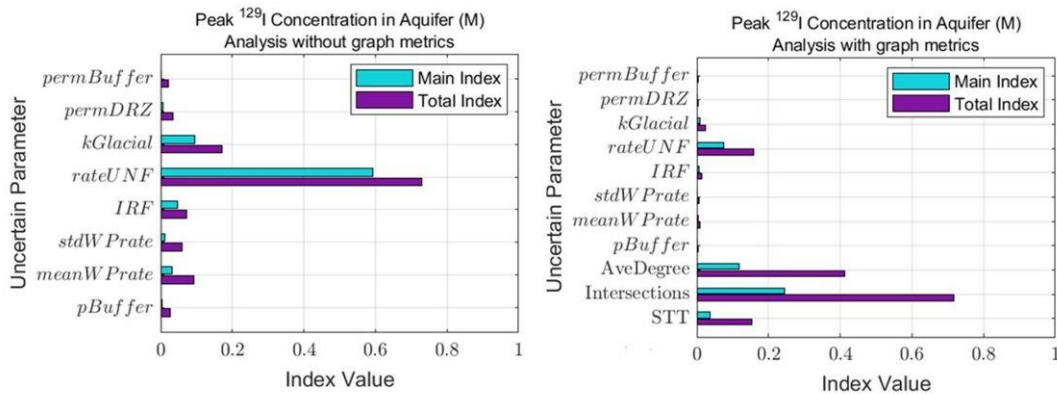
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CRYSTALLINE REFERENCE CASE: MAXIMUM ANALYSIS WITH PCE



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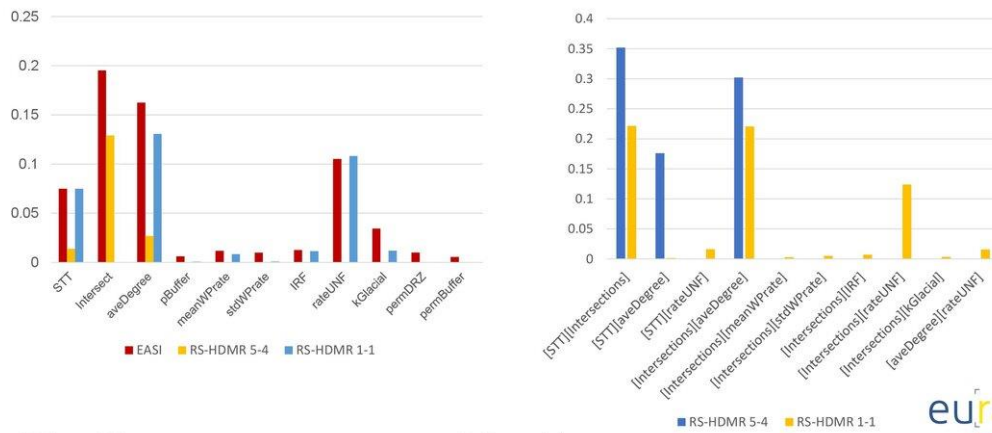
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CRYSTALLINE REFERENCE CASE: MAXIMUM ANALYSIS WITH RS-HDMR



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CRYSTALLINE REFERENCE CASE: VARIANCE-BASED ANALYSIS OUTCOME

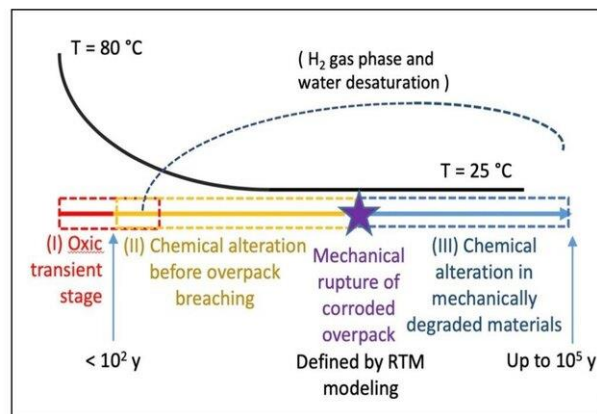
- Clear dominance of graph metrics (if taken into account).
- Most influential epistemic parameter: rateUNF.
- Comparable (low) influences of kGlacial and meanWPrate.
- Considerable dependency on choice of polynomial orders.
- Second-order analysis shows relevant interactions only between graph metrics.
- **Caution: metamodeling is inadequate for aleatory influence of graph metrics**

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PRACTICAL EXAMPLE 3: REACTIVE TRANSPORT IN A GRANITE REPOSITORY (UDC)



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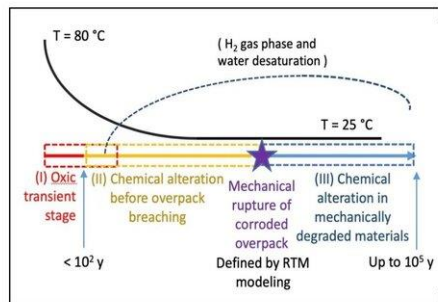
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REACTIVE TRANSPORT MODEL: DESCRIPTION

- Calculation of geochemical and temperature evolution
- Three time periods:
 - Period I: oxidizing unsaturated conditions
 - Period II starts when the buffer is fully saturated
 - Chemical interactions before canister breaching
 - Nonisothermal
 - Anoxic canister corrosion
 - Interactions of corrosion products and buffer material
 - Period III starts after canister failure
 - Glass dissolution & interactions of glass with corrosion products
- Simulation time horizon: 5·10⁴ years

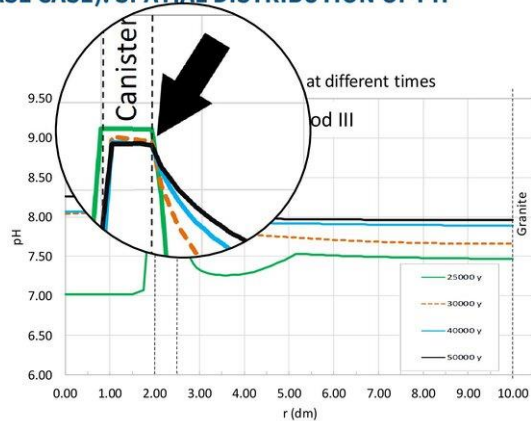
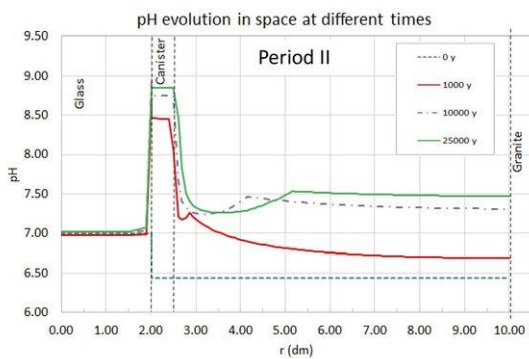


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REACTIVE TRANSPORT MODEL RESULTS (BASE CASE): SPATIAL DISTRIBUTION OF PH



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REACTIVE TRANSPORT MODEL: PLANNING OF PROBABILISTIC ANALYSIS

- Reactive transport simulations were performed on a HPC infrastructure (FinisTerra II cluster from the Galician Supercomputing Center, CESGA, www.cesga.es)
- 27600 simulations
- Output quantity: pH at the canister/bentonite interface (CBI) after $5 \cdot 10^4$ years
- Evaluation with VARS and CUSUNORO

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REACTIVE TRANSPORT MODEL: PARAMETERS

Parameter	Name	Range	PDF	Units
CR	Corrosion rate	0.1 – 10	Uniform	$\mu\text{m/y}$
De	Bentonite dif	0.0631 – 0.631	Log-uniform	dm^2/y
Q ^{gra}	Granite flow	0.01 – 0.1	Log-uniform	L/y
Fe ^{ksel}	Fe selectivity	0.001 – 1	Log-uniform	-
Log(K)	K magnetite	-8.56 – -4.56	Uniform	-

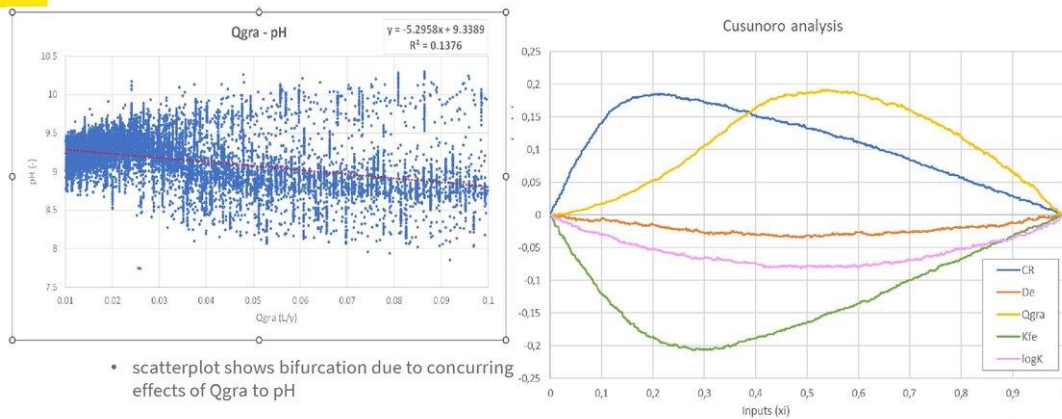
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REACTIVE TRANSPORT MODEL: CUSUNORO ANALYSIS



- scatterplot shows bifurcation due to concurring effects of Qgra to pH

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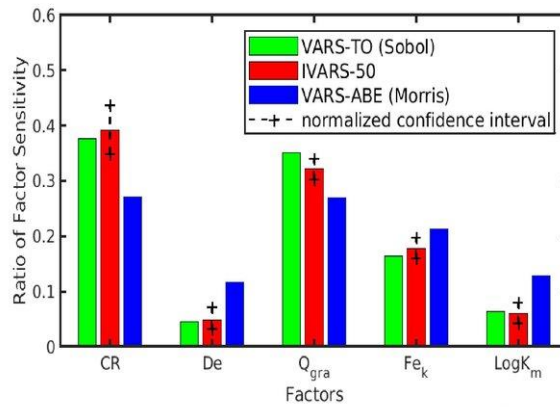
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REACTIVE TRANSPORT MODEL: VARS ANALYSIS

- **Three VARS measures**
 - VARS-TO corresponds to total-order Sobol indices
 - VARS-ABE is the mean local influence (derivative)
 - IVARS-50 is "something in between"
- **VARS-TO and IVARS-50 are very similar**
- **VARS-ABE looks more balanced**
- **In contrast to CUSUNORO**
 - logK seems to have a higher influence than De
 - the influence of Kfe is calculated significantly lower than that of CR and Qgra



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SUMMARY AND CONCLUSIONS.

- Level 1 - Inter has ruinarum varietates a Nisibi quam tuebatur accitus Vrsicinus, cui nos obsecuturos iunxerat imperiale praeceptum, dispicere litis exitialis certamina cogebatur abnuens et reclamans.
- Level 2 - adulatorum oblatrantibus turmis, bellicosus sane milesque semper et militum ductor sed forensibus iurgiis longe discretus, qui metu sui discriminis anxius cum accusatores quaesitoresque subditivos.
- Level 3 - Sibi consociatos ex isdem foveis cerneret emergentes, quae clam palamve agitabantur, occultis Constantium litteris edocebat inplorans subsidia, quorum metu tumor notissimus Caesaris.
- Level 4 - Inter has ruinarum varietates a Nisibi quam tuebatur accitus Vrsicinus, cui nos obsecuturos iunxerat imperiale praeceptum

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BIBLIOGRAPHY AND/OR REFERENCES.

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Appendix F. Slides of lecture B.1

B.1 Characterization and significance of uncertainties related to the waste inventory

Prepared by Attila Baksay, TSENERCON



CHARACTERIZATION AND SIGNIFICANCE OF UNCERTAINTIES RELATED TO THE WASTE INVENTORY

An Bielen (SCK CEN), Sylvia De Gregorio y Robledo (ENRESA), Arutas Plukis (FTMC), Attila Baksay (TS ENERCON) • WP 10



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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KEY WORDS.

- Radioactive waste management,
- Sources of uncertainties of radioactive waste inventory,
- Significance of uncertainties in waste inventory

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Keywords
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CONTENTS

- Introduction
- Radioactive Waste Management and Inventory
 - Radioactive waste management
 - Inventory and its changes during the waste management process
- Uncertainties in Radioactive Waste Inventory
 - Sources of uncertainty in waste inventory
 - Significance and characterization of uncertainties affecting radioactive waste inventory
- Examples of characterization and significance of uncertainties related to radioactive waste inventory
- Summary

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LEARNING OUTCOMES.

After the completion of this training unit/lesson, the students/participants should be able to

- list the elements of radioactive waste management and inventory,
- identify the uncertainties that affect the radioactive waste inventory,
- define the significance of the uncertainties related to radioactive waste inventory,
- assess their own radioactive waste management program from the inventory point of view,
- give examples on the characterization and significance of uncertainties related to the radioactive waste inventory.

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INTRODUCTION

- Problem of radioactive waste starts with its inventory
- Planning steps, shielding, management routes, possible storage and disposal options have roots back to inventory
- Inventory provides the source term for all safety assessment dealing with radioactive waste management
- There are many uncertainties related to the inventory of radioactive waste
- Introduction of radioactive waste and its management
- Identification of points where uncertainties have clear effect on the inventory

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INTRODUCTION -2

- Introduction of uncertainties related to
 - Radiological properties
 - Physico-chemical properties
 - Waste processing (pre-treatment, treatment, conditioning)
 - Long term behavior of radioactive waste
- Significance of these uncertainties
- What are the plans to reduce uncertainties, what are their foreseen evolution?
- Examples to illustrate the significance of uncertainties regarding their significance and characterization
- Presented information is based on IAEA⁽¹⁾ and EURAD^(2,3) sources

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RADIOACTIVE WASTE MANAGEMENT AND INVENTORY



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Radioactive Waste Management and Inventory



WHERE IS RADIOACTIVE WASTE COMING FROM?

- Material for which no further use is foreseen that contains, or is contaminated with, radionuclides at activity concentrations greater than clearance levels as established by the regulatory body.¹
- Operation and Decommissioning of Nuclear Power Plants
- Fuel Cycle Facilities
- Institutional Waste from Industrial, Medical and Research Application
- Legacy waste

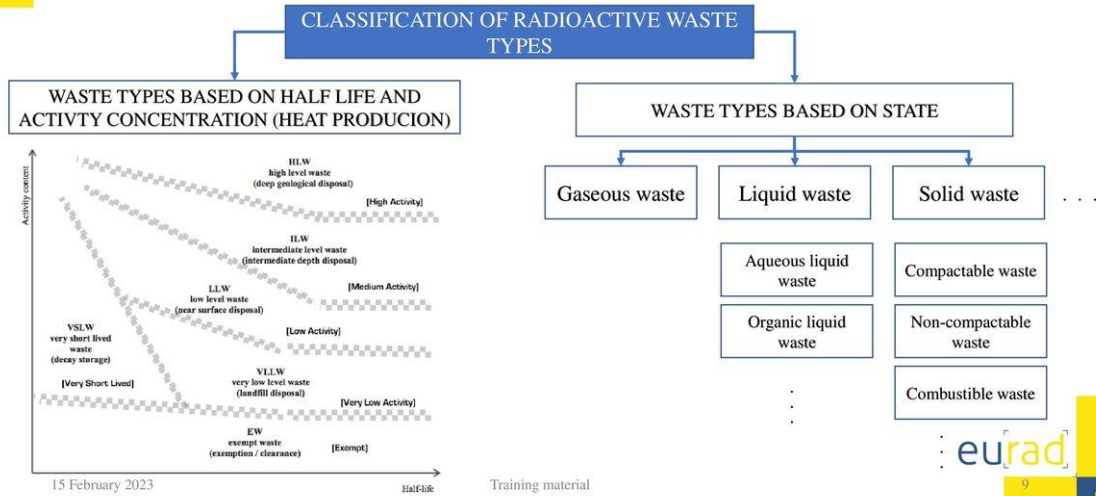


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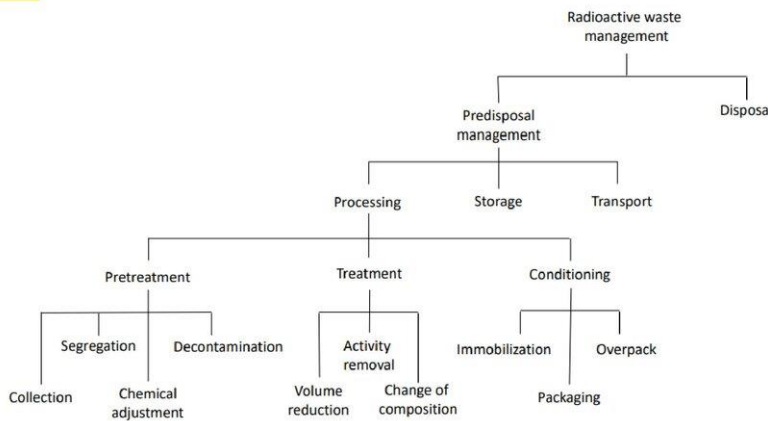




RADIOACTIVE WASTE TYPES



RADIOACTIVE WASTE MANAGEMENT



PREDISPOSAL MANAGEMENT OF RADIOACTIVE WASTE:

Any waste management steps carried out prior disposal, such as:

- Pre-treatment
- Treatment
- Conditioning
- Storage
- Transport

IAEA Glossary 2018

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WASTE ACCEPTANCE CRITERIA (WAC)

- Quantitative or qualitative criteria specified by the regulatory body, or specified by an operator and approved by the regulatory body, for the waste form and waste package to be accepted by the operator of a waste management facility. ⁽¹⁾
 - Waste acceptance criteria specify the radiological, mechanical, physical, chemical and biological characteristics of waste packages and unpackaged waste.
 - Waste acceptance criteria might include, for example, restrictions on the activity concentration or total activity of particular radionuclides (or types of radionuclide) in the waste, on their heat output or on the properties of the waste form or of the waste package.
 - Waste acceptance criteria are based on the safety case for the facility or are included in the safety case as part of the operational limits and conditions and controls.



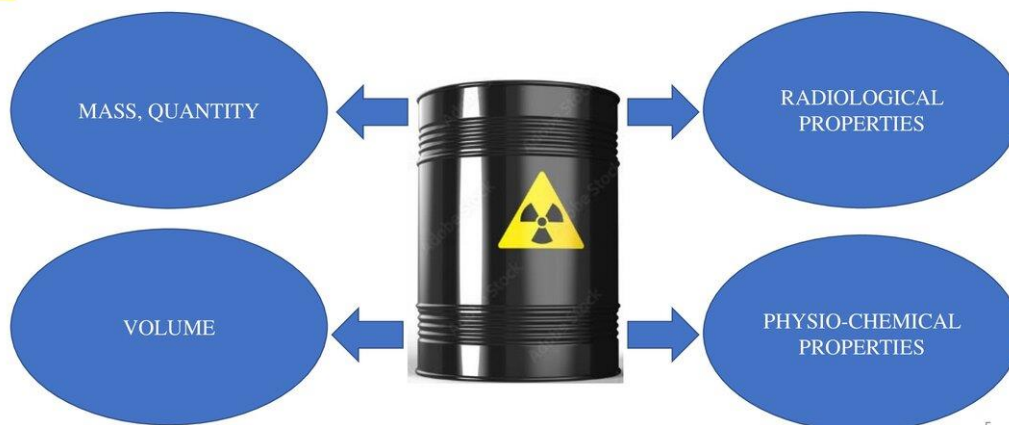
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Radioactive Waste Management and Inventory

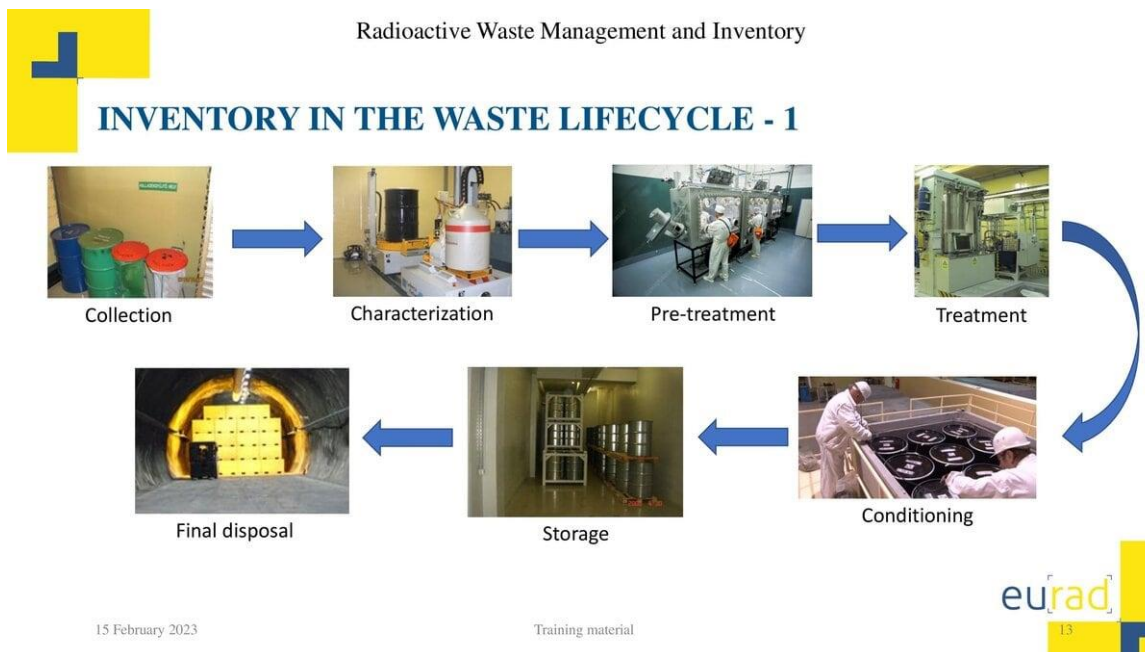
WHAT IS WASTE INVENTORY?



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Radioactive Waste Management and Inventory

INVENTORY IN THE WASTE LIFECYCLE - 2

	Radiological properties	Volume	Mass	Physio-chemical properties
Collection				
Characterization	Difficult-to-measure isotopes			
Pre-treatment	Decay	Volume change	Mass change	Change
Treatment	Decay	Volume change	Mass change	Change
Conditioning	Decay	Volume change	Mass change	Change
Storage	Decay			Possible change
Disposal	Decay		Mass change	Possible change

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UNCERTAINTY IN RADIOACTIVE WASTE INVENTORY



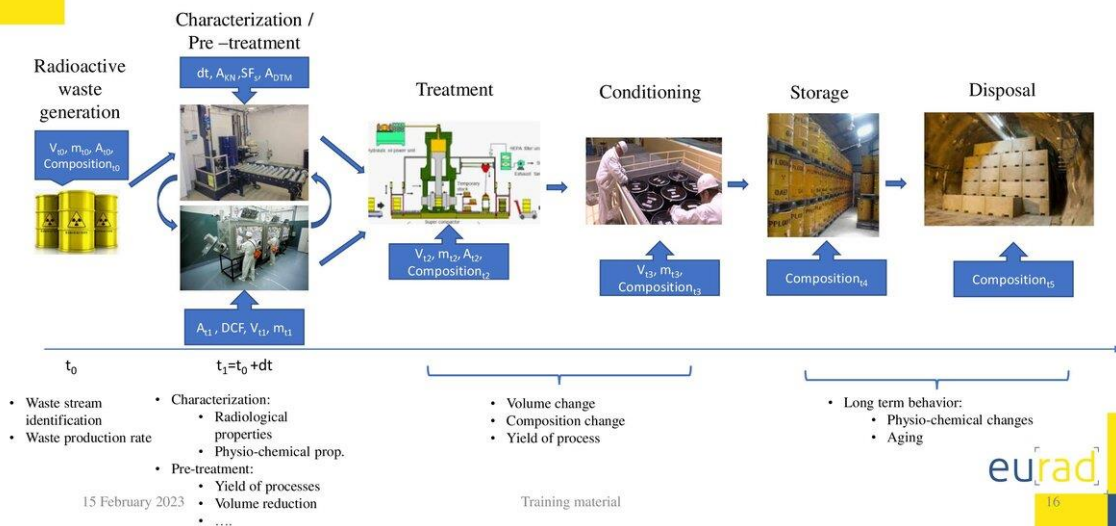
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Uncertainties in Radioactive Waste Inventory



SOURCES OF UNCERTAINTIES IN INVENTORY



Uncertainty in Radioactive Waste Inventory



LIST OF UNCERTAINTIES ASSOCIATED WITH WASTE INVENTORY⁽²⁾

Topical area	Associated uncertainties
Radiological properties	List of critical radionuclides Radionuclide activity Scaling factor
Physico-chemical properties	Cellulose content Behavior of cementitious waste forms Chemical composition Thermal reactivity Swelling Volume, mass Voids
Waste stream identification & management route	Waste streams & associated waste forms WAC's Limited knowledge Not defined routes
Pre-treatment, treatment and conditioning processes	Pre-treatment techniques Treatment techniques Conditioning techniques
Long-term behaviour	Physico-chemical conditions in the storage or disposal facility Waste form behaviour and ageing in storage or disposal conditions Storage time

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Uncertainty in Radioactive Waste Inventory



RADIOLOGICAL PROPERTIES: RADIONUCLIDE ACTIVITY

- **Depending on the focus, the list of the radionuclides can be different**
 - Operational phase of facilities:
 - short lived gamma emitters, neutron emitters (⁶⁰Co, ¹³⁷Cs, ²⁵²Cf....),
 - isotopes of mobile, volatile elements (³H, ¹⁴C, ¹³¹I, ²²²Rn....)
 - fissile isotopes (²³⁵U, ²³⁹Pu, ...)
 - Post closure phase of disposal facilities:
 - Long lived, mobile isotopes add to the list (³⁶Cl, ⁷⁹Se, ¹³⁵Cs, ...)
- **A variety of measurement and calculation methods can be applied to estimate the activity levels of radionuclides enclosed in a waste package. Uncertainties are associated with each of these methods.**
- **Radiological waste characterization is often based on the determination of the key nuclides gamma-ray emitting radionuclides by a measurement and the estimation of the DTM alpha and beta emitter radionuclides via SF method.**

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RADIONUCLIDE ACTIVITY - SIGNIFICANCE

- **Uncertainties related to radionuclide activity including scaling factor**
 - Proper characterization of RW
 - Proper classification
 - Key information with regard to the dose rate
 - Appropriate and optimal waste management route
 - Underestimated dose rates
 - Limiting choice to suboptimal waste management scenario

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Training material

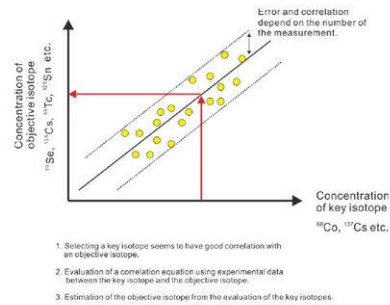


RADIOLOGICAL PROPERTIES: SCALING FACTORS

- **Scaling Factors are determined by detailed multi-step analysis:**
 - Representative samples from the waste stream
 - Radiochemical analysis, determining activity concentrations for gamma emitters as well as beta- and alpha emitters
 - Correlations between activity concentrations of gamma emitters and beta- and alpha emitters

$$AC_{DTM} = b \cdot AC_{KN}^m$$

- **Radiological waste characterization is often based on the determination of the easy-to-measure (ETM) gamma-ray emitting radionuclides, called Key-Nuclides (KN).**
- **Determination of alpha and beta emitter (commonly referred as difficult-to-measure (DTM) radionuclides) activities is based on scaling factors (SF)**



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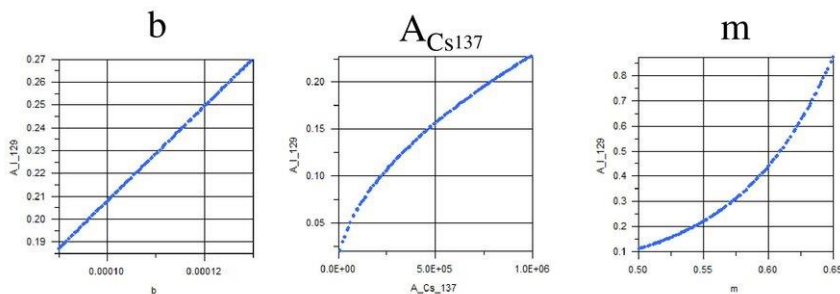




SCALING FACTORS - SIGNIFICANCE

- Example: Effect of the uncertainties in SF parameters and ¹³⁷Cs activity concentration on the activity concentration of ¹²⁹I

$$AC_{DTM} = b \cdot AC_{KN}^m$$



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CHARACTERIZATION OF UNCERTAINTIES OF RADIOLOGICAL PROPERTIES

- **Uncertainties related to radionuclide activity including scaling factor**
 - Should be implemented at each stage of their management
 - Focus on technical improvements
 - Improve statistical approaches
 - Scaling factors -> narrowing the uncertainty
 - Development of adequate methodology
 - Measurement uncertainties
 - Model uncertainties
- **Evolution:**
 - Decrease of uncertainties because of:
 - Operational experience, which will give more insight and more specific knowledge on the behavior of waste and its activities.
 - Improvement of characterization quality.

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Training material





Uncertainty in Radioactive Waste Inventory

UNCERTAINTIES IN PHYSICO-CHEMICAL PROPERTIES

- Physico-chemical properties of RW packages play a role in the conformity with the storage and/or the disposal facility.

Properties related to the RN release from the waste form	Behavior of the waste in relation to its conditioning matrix	Behavior of surrounding disposal components
<ul style="list-style-type: none"> • content of corrosive materials • biodegradation • gas production • free liquid content • leachability • homogeneity 	<ul style="list-style-type: none"> • effects of heat and radiation • flammability • physical form • mechanical strength • content of corrosive materials • biodegradation • gas production 	<ul style="list-style-type: none"> • effects of heat and radiation • physical form • mechanical strength

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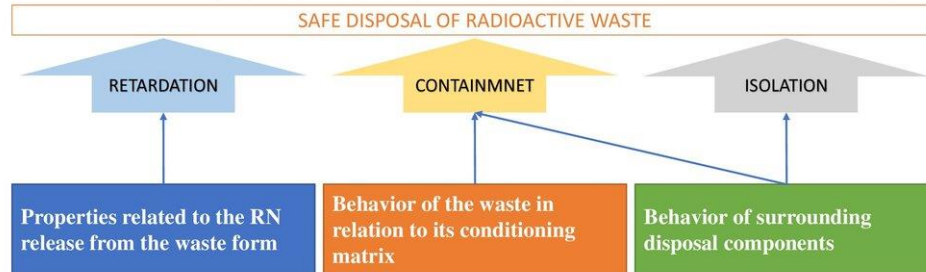
Training material



Uncertainty in Radioactive Waste Inventory

PHYSICO-CHEMICAL PROPERTIES - SIGNIFICANCE

- The purpose of the RW management is to guarantee a safe storage and final disposal
 - Minimum or no radioactive release to the environment
 - Integrity of the waste matrix
 - Integrity of the waste package in disposal environment
- Connection between physico-chemical properties and safety functions of radioactive waste disposal



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Uncertainty in Radioactive Waste Inventory

CHARACTERIZATION OF UNCERTAINTIES IN PHYSICO-CHEMICAL PROPERTIES

- **Uncertainties related to chemical composition with a special attention to organic content**
 - Research and investments regarding the characterization of chemical composition
 - Mass of overall materials and additives is uncertain
 - WAC
 - Internal safety reviews
 - Revision of legislation
 - More stringent final WAC's
 - Limited accuracy of measurements
 - Waste generation history
 - Evolution of the waste

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Uncertainty in Radioactive Waste Inventory

EVOLUTION OF UNCERTAINTIES IN PHYSICO-CHEMICAL PROPERTIES

- **Recent answers show mixed picture on the expected evolution of uncertainties in physico-chemical properties**
- **Positive view in improvements will help in reducing uncertainties:**
 - Technical improvement
 - Better performance laboratories
 - Development in QA/QC
- **Conservative views:**
 - A lot of uncertainties rely on speculation
 - Difficult, what to measure, how to measure and what is the goal
 - Improvements in safety assessments are needed

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PRE-TREATMENT, TREATMENT AND CONDITIONING PROCESSES

- Phases of the radioactive waste processing activities are dedicated to transform raw RW to a waste form that is capable to withstand the conditions of long-term storage and/or final disposal.
- Pre-treatment prepares the waste for further processing and may include sorting and separating different waste materials or waste types, as well as size/volume reduction or shredding to optimize treatment and disposal.
- The aim of waste treatment processes is to transform the waste into a stable and solid physico-chemical form, and to reduce the volume of the waste.
- Conditioning puts the waste in a safe, stable, and manageable form for transport, storage and disposal.

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UNCERTAINTIES IN PRE-TREATMENT, TREATMENT AND CONDITIONING PROCESSES - SIGNIFICANCE

- Waste which is not properly sorted, could have in the end an impact on the physico-chemical properties and volume of the waste.
- Uncertainties related to other pre-treatment processes could have a direct effect on the radionuclide content and physico-chemical form of the waste, as well as an influence on the eventual volume of the waste that shall be produced.
- Uncertainties related to treatment processes could result in more hazardous waste forms from radiation protection point of view.
- Uncertainties associated with conditioning processes could have an impact on the content of radionuclides and dose rate, due to the type of shielding the package offers. Conditioning also has a direct effect on physico-chemical properties and can, in some cases, lead to new uncertainties regarding these properties

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Uncertainty in Radioactive Waste Inventory

CHARACTERIZATION EVOLUTION EVOLUTION OF UNCERTAINTIES IN PRE-TREATMENT, TREATMENT AND CONDITIONING PROCESSES

- **Technological development**

- Decontamination strategy and technology developments
- Characterization technology developments
- QA/QC developments

- **Evolution**

- Uncertainties will be reduced thanks to development of new techniques

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Uncertainty in Radioactive Waste Inventory

LONG-TERM BEHAVIOR OF RADIOACTIVE WASTE

- **Physico-chemical conditions in the storage or disposal facility:**

- The long-term behavior of the waste package and waste matrix in the surface facility or underground conditions due to uncertainties in the facility, such as pH, presence of aggressive species (chlorides, sulphates,...), mechanical stresses, etc.

- **Degradation rate of the waste:**

- Uncertainty in changes in matrix, degradation rate and integrity of the waste form/package
- Uncertainty on release rates of radionuclides and aggressive species in the waste form
- Uncertainty on waste-induced disturbances of other disposal system components

- **Storage time:**

- Storage facilities are designed for specific classes of waste packages and therefore their design and implementation must take into account all the operational risks related to these classes of waste including those associated with extended storage times. The storage may be short term to allow for radioactive decay or long-term until the waste can be safely transferred to a suitable disposal site

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Uncertainty in Radioactive Waste Inventory

LONG-TERM BEHAVIOR OF RADIOACTIVE WASTE - SIGNIFICANCE

- **Physico-chemical conditions in the storage or disposal facility:**
 - Uncertainty in disposal behaviors could produce reactions with the waste form, or result in corrosion, swelling, etc. of the matrix. Influence on the properties of the waste form – degradation rate.
 - Uncertainties in the storage or disposal facility conditions will impact all parameters related to the long-term waste package behavior and can result in leaching out radionuclides, chemicals, by-products, etc. into the environment.
- **Waste form behavior, degradation rate**
 - Reduction of the stability of the waste form will in turn affect those parameters that are responsible for the migration rate of the contaminants, evolving in uncertainty in which way, rate and extent the radionuclides will be released from the facility.
- **Storage time**
 - Uncertainties associated with storage time might arise and may affect the uncertainties on the waste form characteristics due to ageing processes or the occurrence of events like incidents or accidents.

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Uncertainty in Radioactive Waste Inventory

CHARACTERIZATION OF UNCERTAINTIES IN LONG-TERM BEHAVIOR OF RADIOACTIVE WASTE

- **Uncertainties related to physico-chemical conditions in the storage or disposal facility**
 - Information on
 - Micro-climate indoors
 - Barriers
 - Surroundings
 - Collecting data from experiments
 - Modelling

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Uncertainty in Radioactive Waste Inventory

EVOLUTION OF UNCERTAINTIES IN LONG-TERM BEHAVIOR OF RADIOACTIVE WASTE

- New information about the characteristics of the disposal site together with the development and implementation of the disposal program may be responsible for the reduction of this uncertainty.
- The progress of siting processes, decisions on possible type(s) of host rock and the implementation of the uncertainty in the safety assessment will decrease the uncertainty over time.

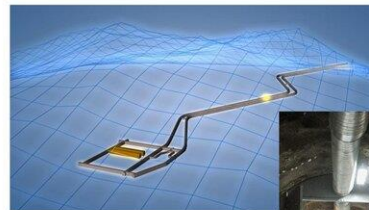
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EXAMPLE 1: STOCHASTIC ISOTOPE INVENTORY FOR BÁTÁAPÁTI

- Bataapáti National Radioactive Waste Repository
- Goal:
 - Input data for:
 - Post closure safety assessment (GoldSim)
 - Dose assessment for operational phase
 - Assessment for later measurement data
- Used data:
 - Waste stream- and isotope specific measured data
 - Generation rate for different waste streams
- Results:
 - Activity concentrations for the waste streams during and after the operational period of the NPP

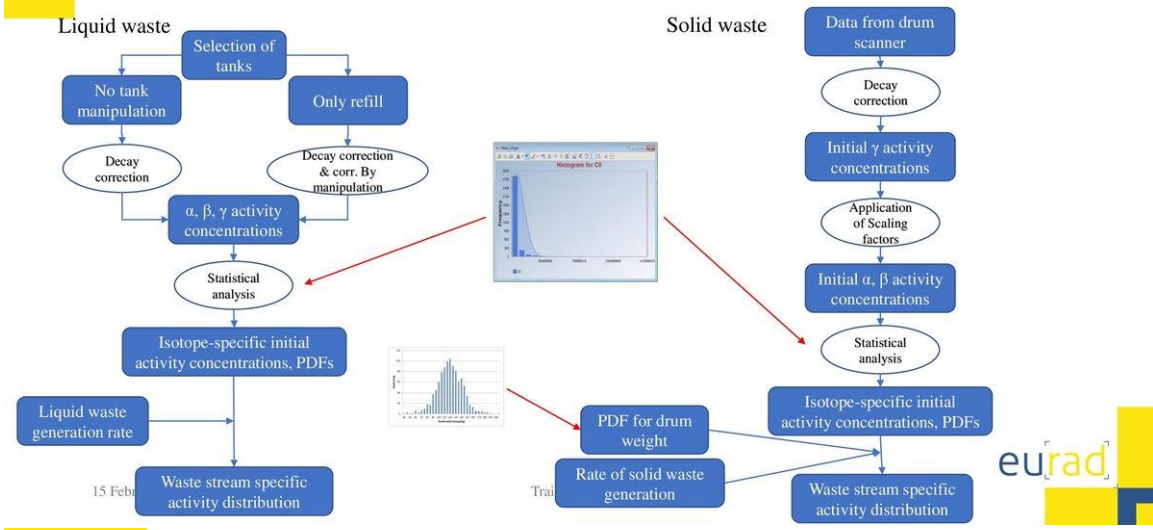


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DERIVATION OF PDFS FOR ACTIVITY CONCENTRATIONS



STOCHASTIC SOURCE TERM

- Software: GoldSim
- Capable of stochastic modelling
- Using pdfs and time series

Statistics for AC_OilyDrum(Cs_137)

Min	1%	5%	10%	25%	50%	75%	90%	95%	99%	100%
Value	1.0e+02	1.0e+03	1.0e+04	1.0e+05	1.0e+06	1.0e+07	1.0e+08	1.0e+09	1.0e+10	1.0e+11

Statistics for AC_ActiveFuel(Pl_3)

Min	1%	5%	10%	25%	50%	75%	90%	95%	99%	100%
Value	1.0e+06	1.0e+07	1.0e+08	1.0e+09	1.0e+10	1.0e+11	1.0e+12	1.0e+13	1.0e+14	1.0e+15

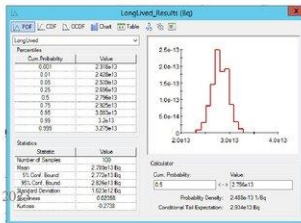
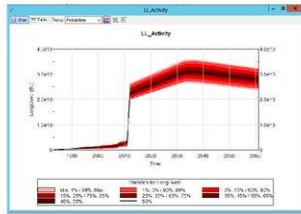
Statistics for AC_ActiveFuel(Pl_5)

Min	1%	5%	10%	25%	50%	75%	90%	95%	99%	100%
Value	1.0e+06	1.0e+07	1.0e+08	1.0e+09	1.0e+10	1.0e+11	1.0e+12	1.0e+13	1.0e+14	1.0e+15



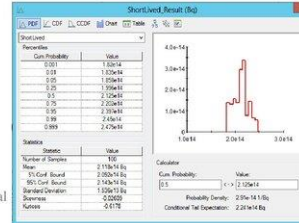
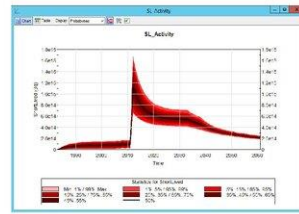
RESULTS OF THE MODELLING

Activity of long-lived isotopes



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Activity of short lived isotopes



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EXAMPLE 2: INCIDENT IN A TREATMENT FACILITY

- An example for the significance of uncertainty in physico-chemical composition of radioactive waste
- Incident took place in Radioactive Waste Treatment Facility
- INES 2 incident in 2013
- Drums were opened in order to sort and compact the waste
- Expected waste form:
 - Am on electroplated steel discs
 - Paper wipes used in laboratory

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EXAMPLE 2: INCIDENT IN A TREATMENT FACILITY -2

- Waste contained significant amount of powder, dried salt that contained ^{241}Am , unknown
- Powder got airborne during the compaction of the waste
- Readings taken following the incident showed the worker's hands, clothing, and the waste compactor room were contaminated.
- According to whole-body measurements taken on 12 December 2013, the amounts of incorporated ^{241}Am for the three workers were 0.9 kBq, 1.8 kBq and 9.0 kBq.

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EXAMPLE 3: RADIOLOGICAL RELEASE FROM WIPP -1

- Waste Isolation Pilot Plant in Carlsbad, NM, USA
- Disposal of defense-generated TRU waste from DOE sites
- Waste is disposed of in a set of panels located nearly one-half mile below the surface in a deep geologic salt bed formed 250 million years ago
- Two accidents:
 - A salt haul truck caught fire 05.02.2014
 - Radiological Release Event on 14.02.2014



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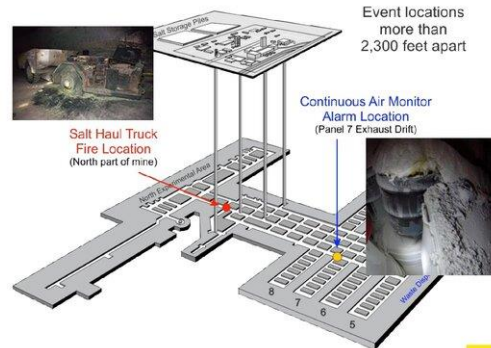
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EXAMPLE 3: RADIOLOGICAL RELEASE FROM WIPP -2 (6)

- Phase II of the investigation focused on the cause of the radiological release, with the AIB concluding that the release was caused by an exothermic reaction involving the mixture of organic materials and nitrate salts in one drum that was processed at Los Alamos National Laboratory in December 2013
- Example of effect of uncertainty in conditioning and long-term changes in chemical form



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EXAMPLE 4: OPTIMIZATION FOR DISPOSAL 1.

- Safety upgrade of RWTDF
- Old RADON type repository
- One of the major goal was to achieve disposal volume gain
- Optimized usage of waste containers
- Clear effects of uncertainties in
 - waste volume,
 - pre-treatment, treatment and conditioning
- It was difficult to estimate the final disposal volume, the result was not so satisfying



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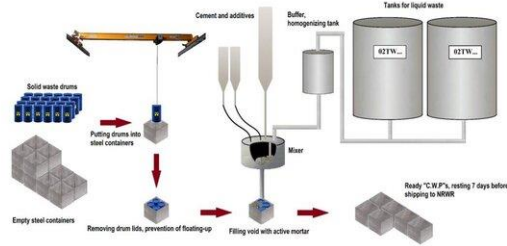
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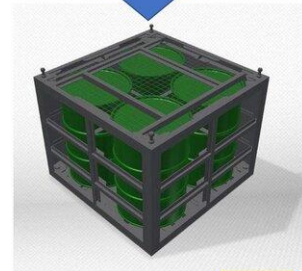
EXAMPLE 5: OPTIMIZATION FOR DISPOSAL 2.

- Geological and economical reasons an optimization project had been carried in Paks NPP
- By changing the pre-treatment, treatment and conditioning methodologies significant volume reduction became possible



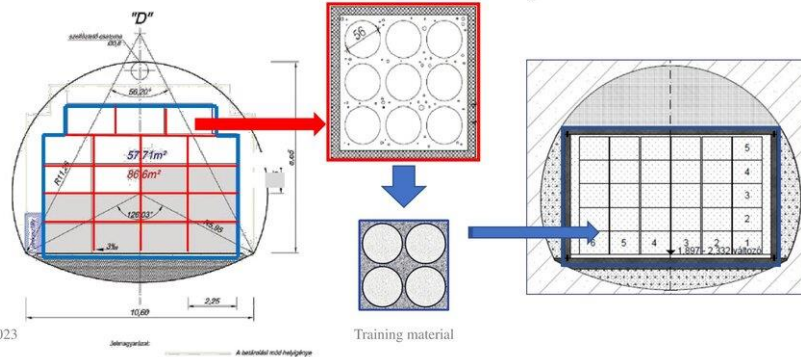
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CHANGE IN THE WASTE PACKAGE AND IN THE DISPOSAL CONCEPT

- Because of the change of the disposal strategy, radiation protection became significantly more important.
- Uncertainties of the source term became more important



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SUMMARY AND CONCLUSIONS.

- Basic information about radioactive waste management was delivered
- Uncertainties discussed:
 - Radiological properties:
 - Physico-chemical properties
 - Waste processing
 - Long term behavior of radioactive waste
- Significance of uncertainties in inventory
 - Safety relevancies (Safety assessments)
 - Role in the optimization
- Characterization
 - Better technologies, methodologies under development
 - Uncertainties are planned to be diminished

HELP NOTE.
Summary and/or conclusions.

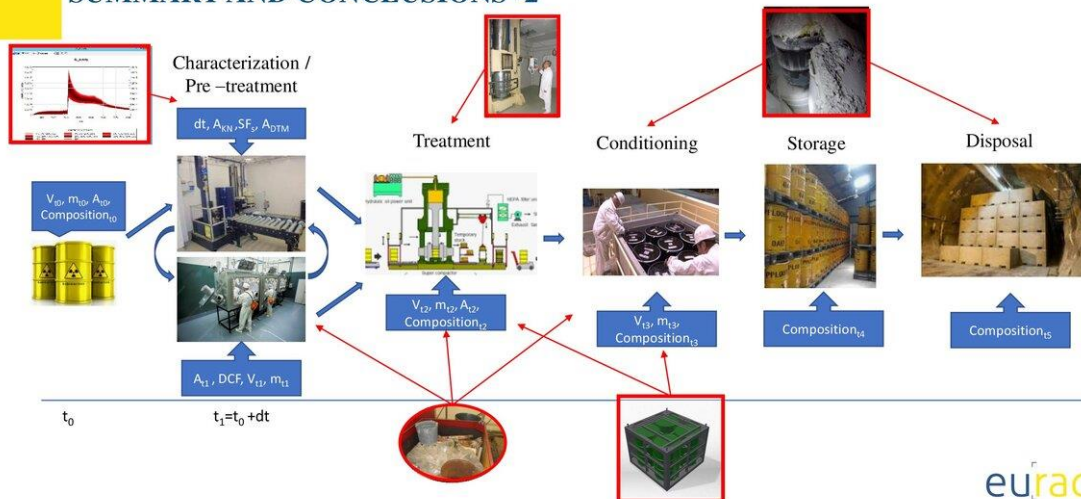
- List of key points in the lesson
- The purpose of the summary is to help the learner understand the lesson
- Organize the ideas in the summary from the most relevant to the least important
- Do not literally copy the same text when summarizing.
- Make sure that the writing is clear and concise
- Plan, write, review and edit your product.

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SUMMARY AND CONCLUSIONS -2



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BIBLIOGRAPHY AND/OR REFERENCES.

- ¹: IAEA Radioactive Waste Management Glossary, 2003 Edition
- ²: EURAD Milestone 92: UMAN Preliminary list of uncertainties from Subtask 3.2 as input to Subtask 4.2, WP 10
- ³: EURAD Deliverable 10.6: Views of the different actors on the identification, characterization and potential significance of uncertainties on waste inventory and on the impact of predisposal steps, WP 10
- ⁴: P. Zagyvai: Examination of physico-chemical properties Am-241 pollution, Rad.protection Training, 2014, Hajdúszoboszló
- ⁵:<https://www.nucnet.org/news/hungary-contamination-incident-gets-provisional-ines-level-2-rating>
- ⁶: U.S. DoE Office of Environmental Management: Accident Investigation Report Phase 2 Radiological Release Event at the Waste Isolation Pilot Plant, February 14, 2014; April 2015

Date

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Appendix G. Slides of lecture B.2

B.2 Uncertainties in the radiological characterization of spent fuel

Prepared by Peter Schillebeeckx, JRC



UNCERTAINTIES IN THE RADIOLOGICAL CHARACTERIZATION OF SPENT FUEL

P. Schillebeeckx • WP 8 Task 2



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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1



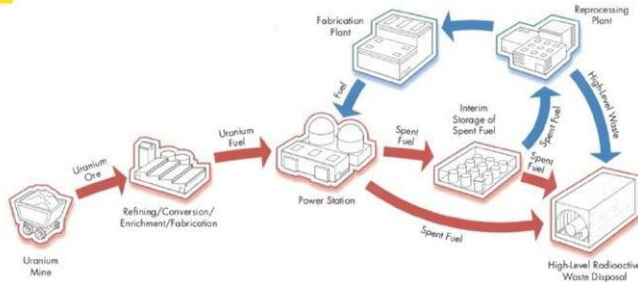
CONTENTS

- Context: spent nuclear fuel management
- Uncertainty evaluation and propagation
 - Definitions and terminology
 - Uncertainty propagation
- Radiological characteristics of spent nuclear fuel (SNF)
 - Radionuclide inventory of SNF
 - Decay heat: example ^{137}Cs
 - Neutron emission
 - Key nuclides
- Depletion calculations
 - Principles
- Validation of depletion codes
 - Radiochemical data (EURAD, ^{36}Cl and ^{129}I)
 - Radiochemical data (SFCOMPO)
 - Calorimeter at CLAB
 - Neutron measurements
- Summary and conclusions
- Material
 - Learning outcomes
 - Keywords
 - Additional bibliography and references

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2

NUCLEAR FUEL CYCLE

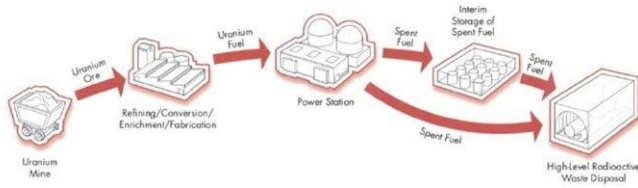


Nuclear fuel cycle
(open or closed)
produces waste

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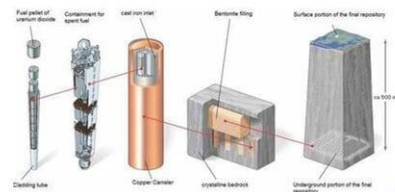
NUCLEAR FUEL CYCLE



Open fuel cycle
Sweden : Forsmark
Finland : Onkalo

Three barriers (SKB):

- canister
- buffer (bentonite clay)
- bedrock



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SPENT NUCLEAR FUEL (SNF) DISPOSAL EXAMPLE: SWEDISH CONCEPT



CLAB
Central Interim Storage Facility
for Spent Nuclear Fuel (SNF)



CLINK
SNF encapsulation plant



SIGRID
SNF carrier



FORSMARK bedrock
SNF disposal

www.skb.com

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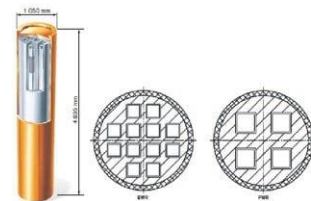
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SNF DISPOSAL EXAMPLE: SWEDISH CONCEPT

- About 1600 canisters
 - 5 m long and 1 m diameter
 - 5 cm thick Cu casing (protect against corrosion)
 - Insert of nodular cast iron (structural integrity)
 - Maximum decay power : 1700 W
- Optimisation of canister loading has a substantial economic and ecological impact
 - Cu- mining and canister production:
 - 1 M€ per extra canister
 - 2 ton Cu per extra canister
 - Repository: tunnel excavation, operation, ...



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OPTIMISING CANISTER LOADING

Reduction of canisters and required disposal volume by

- Improved loading schemes algorithms
e.g. Solans et al., Nucl. Eng. Des. 370 (220) 110897, reduction of 2% in number of canisters
- Improved decay power estimation
Reduction of uncertainty of decay power estimation
(Seidl et al., submitted to Frontiers in Energy research – Nuclear Energy)

One of the main objectives of EURAD WP8 Task 2

“Spent nuclear fuel characterisation and related uncertainty analysis”



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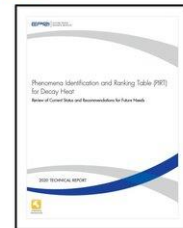
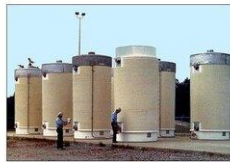


OFFLOADING OF THE FUEL FROM POOL: OPTIMISE REQUIRED COOLING TIME

Electronic Power Research Institute (EPRI)

“Phenomena Identification and Ranking Table (PIRT) for Decay Heat”,
Final Report, July 2020

“..., there is a desire to offload the fuel as early as possible for economic reasons,
since maintaining a Spent Fuel Pool is very costly.”



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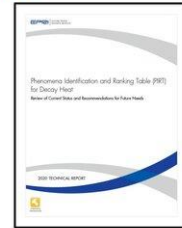
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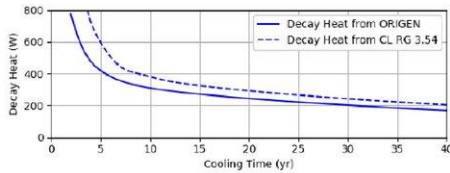
OFFLOADING OF THE FUEL FROM POOL: OPTIMISE REQUIRED COOLING TIME

Electronic Power Research Institute (EPRI)

“Phenomena Identification and Ranking Table (PIRT) for Decay Heat”,
Final Report, July 2020



Difference between regulatory guidelines and SNF specific calculations



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SNF MANAGEMENT : STORAGE, TRANSPORT, REPROCESSING AND DISPOSAL

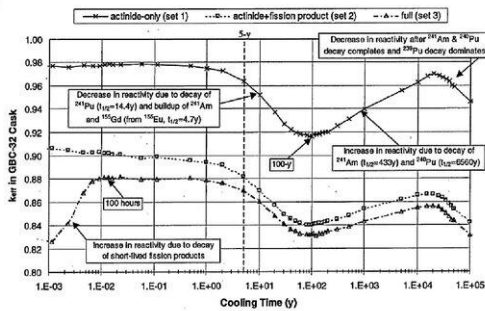


Figure 3 Reactivity behavior in the GRC-32 cask as a function of cooling time for the three classifications of burnup credit (defined in Table 1). The calculations correspond to fuel with 4.0 wt % ²³⁵U initial enrichment that has accumulated 48 GWd/MTU burnup, and include an axial burnup distribution as described in Section 2.1.

Wagner and Parks, NUREG/CR-6781 (January 2003)
“Recommendations on the credit for cooling time in PWR burnup credit analysis”

Nuclear Criticality Safety (reactivity)

NEA: Working Party (WPNCs)

- Must be guaranteed based on reliable methods
- Burn-Up Credit (NEA: Expert group EGBUC)
 - Reactivity decreases as fuel BU proceeds: reduction of fissile material and increase of neutron absorbing fission products (FP)
 - Account for this reactivity decrease for criticality safety assessments

⇒ Requires **inventory of key nuclides** and accurate neutron **absorption cross sections**

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EURAD WP8: SPENT NUCLEAR FUEL CHARACTERIZATION

- **Nuclide inventory** and related **radiological characteristics** of SNF are observables (or measurands) that are determined with an uncertainty
- **Reliable** estimate of the **uncertainty** is required, e.g.
 - to respect the constraint of total decay power < 1700 W
 - to define reliable criticality safety margins



The Orano La Hague reprocessing facility (Photo: Orano)
www.iaea.org



www.skb.com



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UNCERTAINTY EVALUATION AND PROPAGATION

Basis: Guides in metrology published by BIPM <https://www.bipm.org/en/committees/jc/jcgm/publications>

- **GUM: Guide to the expression of Uncertainty in Measurement**
 - **Evaluation of measurements data** (JCGM 100:2008(E))
 - Supplement 1 – Propagation of distribution using a Monte Carlo method (JCGM 101:2008)
 - Supplement 2 – Extension to any number of quantities (JCGM 102:2011)
 - An introduction to the “GUM” and related documents (JCGM 104:2009)
 - Guide to the expression of uncertainty in measurement Part 6: Developing and using measurement models (JCGM GUM-6:2020)
- **VIM: International Vocabulary of Metrology**
 - **VIM – Basic and general concepts and associated terms** (JCGM 200:2012)

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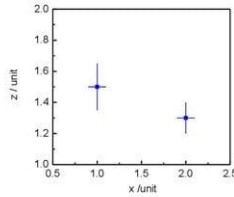


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MEASUREMENT UNCERTAINTY

- **Result** of a measurement is only an approximation or **estimate** of the value of the **measurand (quantity of interest)** and thus is complete only when **accompanied by** a statement of the **uncertainty** of that estimate
- **Uncertainty** of a measurement **parameter**, associated with the result of a measurement, that characterizes the **dispersion** of the values that could reasonably be **attributed** to the **measurand**

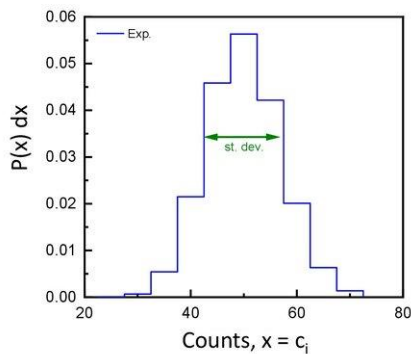


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EXAMPLE: COUNTING EXPERIMENT

c_i : result of a single counting experiment to estimate the counts C

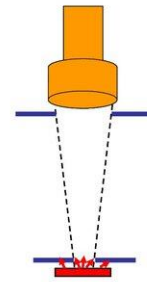


$$\text{Mean : } c = \frac{1}{m} \sum_{i=1}^m c_i$$

$$\text{Variance of mean : } s_c^2 = \frac{s^2(c_i)}{m}$$

$$\text{Variance of a single observation : } s_{c_i}^2 = \frac{1}{m-1} \sum_{i=1}^m (c_i - c)^2$$

Standard uncertainty is expressed as the standard deviation



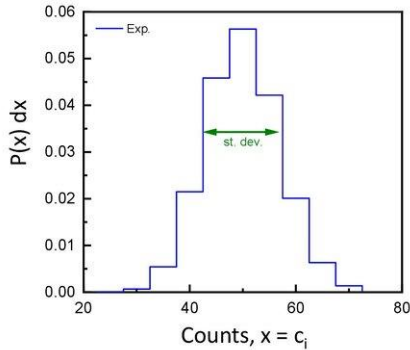
Training material





EXAMPLE: COUNTING EXPERIMENT

c_i : result of a single counting experiment to estimate the counts C



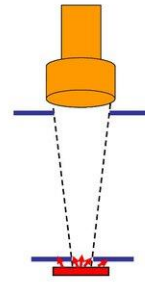
Mean : $c = \frac{1}{m} \sum_{i=1}^m c_i$

Variance of mean : $s_c^2 = \frac{s^2(c_i)}{m}$

Variance of a single observation : $s_{c_i}^2 = \frac{1}{m-1} \sum_{i=1}^m (c_i - c)^2$

Standard uncertainty of c_i : $u_{c_i} = s_{c_i}$

Standard uncertainty of c : $u_c = \frac{s_{c_i}}{\sqrt{m}}$

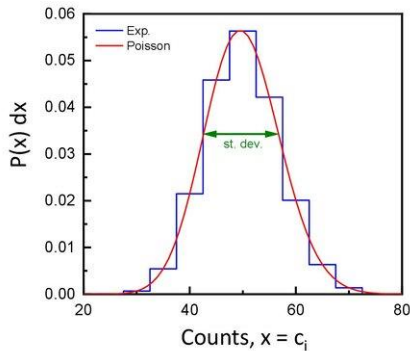


Training material



EXAMPLE: COUNTING EXPERIMENT

c_i : result of a single counting experiment to estimate the counts C



Poisson distribution

$P(c, \mu) = \frac{e^{-\mu} \mu^c}{c!}$

Mean : μ

Variance : $\sigma^2 = \mu$

Standard deviation : $\sigma = \sqrt{\mu}$

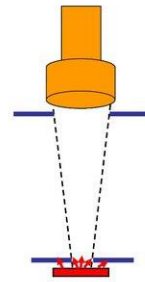
Result of a single counting experiment

Counts : c_i

Uncertainty : $u_{c_i} = \sqrt{c_i}$

$\frac{u_{c_i}}{c_i} = \frac{1}{\sqrt{c_i}}$

Uncertainty due to counting statistics



Training material

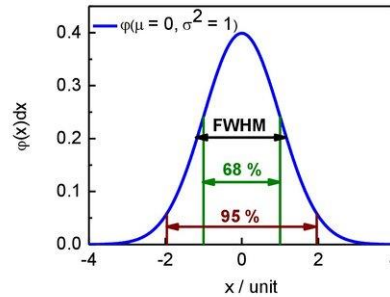




REPORTING OF UNCERTAINTIES

- Standard ($k = 1$) or expanded ($k > 1$) uncertainty (x, u_x) with $u_x = k s_x$
 - Standard uncertainty
 $k = 1 \Rightarrow 0.68 \%$
 - Expanded uncertainty
 $k > 1$
 e.g. $k = 1.96 \Rightarrow 0.95 \%$
 $k = 2.58 \Rightarrow 0.99 \%$
- Reporting example: $x = 10.21$ with $u_x = 0.25$
 - $k = 1$ 10.21 (25) or 10.21 (0.25)
 - $k > 1$ 10.21 \pm 0.25 (specify k , mostly $k = 2$)

Note: physicists mostly report a standard uncertainty ($k = 1$)
 chemists mostly report an expanded with $k = 2$.



Training material



UNCERTAINTY PROPAGATION: INDEPENDENT VARIABLES

(y, u_y) , (b, u_b) and (k, u_k) **independent** input quantities \Rightarrow estimate of (z, u_z)

$$\begin{aligned}
 z &= y + b & u_z^2 &= u_y^2 + u_b^2 \\
 z &= y - b & u_z^2 &= u_y^2 + u_b^2 \\
 z &= k y & u_z^2 &= k^2 u_y^2 + y^2 u_k^2 & \frac{u_z^2}{z^2} &= \frac{u_y^2}{y^2} + \frac{u_k^2}{k^2}
 \end{aligned}$$

(y, u_y) and constant $K \Rightarrow$ estimate of Z

$$z = K y \quad u_z^2 = K^2 u_y^2 \quad \frac{u_z^2}{z^2} = \frac{u_y^2}{y^2}$$

Training material





LINEAR FUNCTION OF INDEPENDENT VARIABLES

Z : **linear function** of independent random variables $X_{i=1,\dots,n}$ with a normal PD with (μ_i, σ_i^2)

$$z = \sum_{i=1}^n c_i X_i \qquad c_i = \left. \frac{\partial f}{\partial X_i} \right|_{\mu_i}$$

⇒ PD of $Z = f(X_i; i, \dots, n)$ is a normal distribution with

- Mean $E(Z) = \mu_z = \sum_{i=1}^n c_i \mu_i$
- Variance $V(z) = \sigma_z^2 = \sum_{i=1}^n c_i^2 \sigma_i^2$

Training material



NON-LINEAR FUNCTION OF INDEPENDENT VARIABLES

Z : **non-linear function** of independent random variables $X_{i=1,\dots,n}$ with a normal PD with (μ_i, σ_i^2)

1st order Taylor development

$$z \approx f(\mu_1, \dots, \mu_n) + \sum_{i=1}^n g_i (x_i - \mu_i) \qquad g_i = \left. \frac{\partial f}{\partial X_i} \right|_{\mu_i}$$

⇒ PD of $Z = f(X_i; i, \dots, n)$ is a normal distribution with

- Mean $E(Z) = \mu_z \approx f(\mu_1, \dots, \mu_n)$
- Variance $V(z) = \sigma_z^2 \approx \sum_{i=1}^n g_i^2 \sigma_i^2$

Training material





SENSITIVITY ANALYSIS AND UNCERTAINTY PROPAGATION

Z : non-linear function of independent random variables $X_{i=1,\dots,n}$ with a normal PD with (μ_i, σ_i^2)

1st order Taylor development

$$z \approx f(\mu_1, \dots, \mu_n) + \sum_{i=1}^n g_i (x_i - \mu_i)$$

$$g_i = \left. \frac{\partial f}{\partial x_i} \right|_{\mu_i}$$

Sensitivity analysis based on partial derivatives:
study how the output Z depends on the variables X_i

⇒ PD of $Z = f(X_i; i, \dots, n)$ is a normal distribution with

• Mean $E(Z) = \mu_z \approx f(\mu_1, \dots, \mu_n)$

• Variance

$$V(z) = \sigma_z^2 \approx \sum_{i=1}^n g_i^2 \sigma_i^2$$

$$\frac{u_z}{z} = R_{x_i} \frac{u_{x_i}}{x_i} \quad R_{x_i} = \frac{g_i}{\frac{z}{x_i}}$$



Training material



SENSITIVITY ANALYSIS AND UNCERTAINTY PROPAGATION: DEPENDENT VARIABLES

Z : non-linear function of dependent random variables $X_{i=1,\dots,n}$ with a normal PD with $(\vec{\mu}, V_x)$

$$z \approx f(\vec{\mu}_x) + G_x (\vec{x} - \vec{\mu}_x)$$

G_x : gradient matrix of f

• Mean

$$\mu_z \approx f(\vec{\mu}_x)$$

$$g_k = \frac{\partial f}{\partial x_k}$$

• Variance

$$V_z \approx G_x V_x G_x^T$$

⇒ basis of General Law of Uncertainty Propagation (GLUP)
(sandwich formula, $V_z = G V_x G^T$)
Based on normal distributions



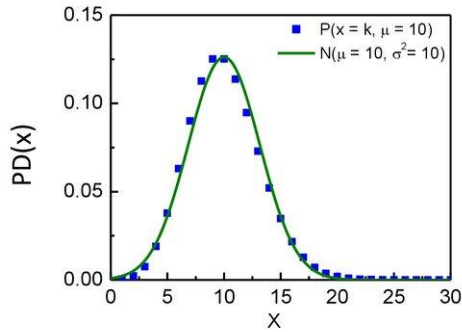
Training material



NORMAL DISTRIBUTION ASSUMPTION

- (1) Poisson distribution to account for uncertainty due to counting statistics

For large μ the distribution approaches a normal distribution



$$\text{Poisson : } P(x = k, \mu) = e^{-\mu} \frac{\mu^k}{k!}$$

$$\text{Normal : } N(x, \mu, \sigma) = \frac{1}{\sigma\sqrt{2\pi}} e^{-\frac{1}{2}\left(\frac{x-\mu}{\sigma}\right)^2}$$

Training material



NORMAL DISTRIBUTION ASSUMPTION

- (1) Poisson distribution to account for uncertainty due to counting statistics

For large μ the distribution approaches a normal distribution

- (2) Central limit theorem (CLT)

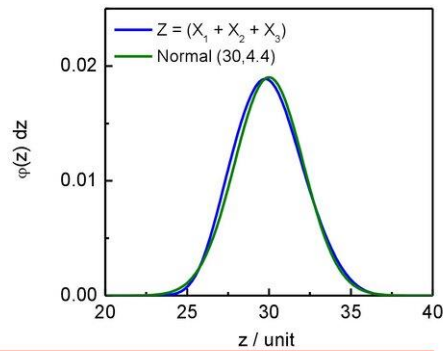
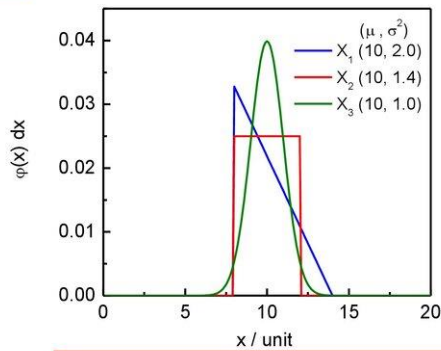
The sum of a large number of independent random variables with a similar distribution (i.e. width) will be approximately normally distributed

Training material





NORMAL DISTRIBUTION ASSUMPTION: CENTRAL LIMIT THEOREM

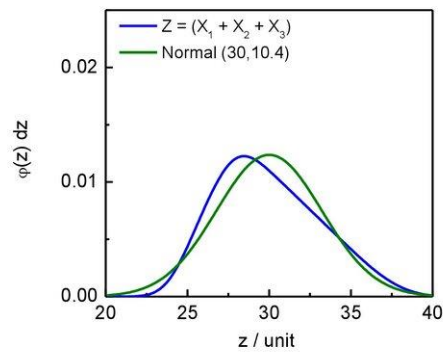
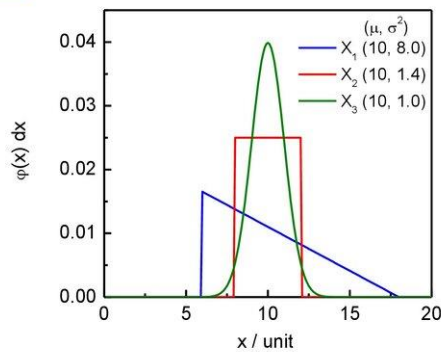


$$Z = \sum_{i=1}^n X_i \text{ normal distribution with } \mu_Z = \sum_{i=1}^n \mu_i \text{ and } \sigma_Z^2 = \sum_{i=1}^n \sigma_i^2$$

Training material



NORMAL DISTRIBUTION ASSUMPTION: CENTRAL LIMIT THEOREM



$\sigma_1 > (\sigma_2 \text{ and } \sigma_3) \Rightarrow$ combination deviates from a normal distribution

Training material





NORMAL DISTRIBUTION ASSUMPTION

- (1) Poisson distribution to account for uncertainty due to counting statistics
For large μ the distribution approaches a normal distribution
- (2) Central limit theorem (CLT)
The sum of a large number of independent random variables with a similar distribution (i.e. width) will be approximately normally distributed
- (3) Principle of maximum entropy (ME)
If only the mean and standard deviation is given, the optimal probability distribution for further inference is the normal distribution

⇒ in most cases normal distribution can be assumed

Training material



ERROR AND UNCERTAINTY

- Measurement error : **difference between two values**
“*result of a measurement minus a true value of the measurand*”
can be + or -
- Measurement uncertainty : **dispersion of a distribution**
“*non-negative parameter characterizing the dispersion of the values being attributed to the measurand*”
always > 0
determined by the width of the PD of the uncertainty component(s)

error ≠ uncertainty

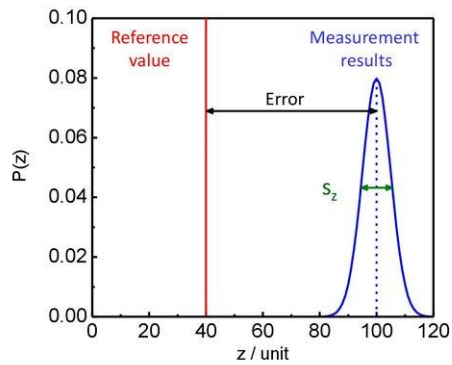
Training material





ERROR AND UNCERTAINTY

z : result of a measurement to estimate the value of quantity Z



- **Measurement error**
Difference between values
+ or -
- **Uncertainty**
Derived from the standard deviation
(width) of a distribution
> 0

Training material



RANDOM AND SYSTEMATIC ERROR

- **Random error**
“Result of a measurement minus the mean that would result from an infinite number of measurements of the same measurand carried out under repeatability conditions.”
- **Systematic error (bias)**
“Mean that would result from an infinite number of measurements of the same measurand carried out under repeatability conditions minus a true value of the measurand.”

Bias: estimate of a systematic error with respect to a reference value of the same measurand

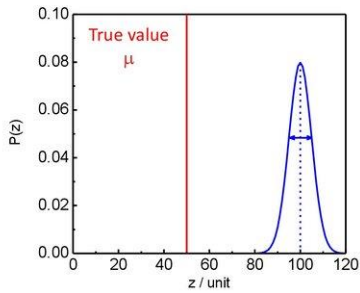
Training material





$(Y_1, \dots, Y_N, B) \Rightarrow$ ESTIMATE OF $Z = Y - B$

- Z : measurand, i.e. quantity of interest, with true value μ
- y_i : result of a single experiment to estimate true value of Z
all measurements in same conditions (e.g. measurement time,...)
- b : correction for background, $z = y - b$



$$y = \frac{1}{m} \sum_{j=1}^m y_j \quad u_y = \frac{s_{y_j}}{\sqrt{m}}$$

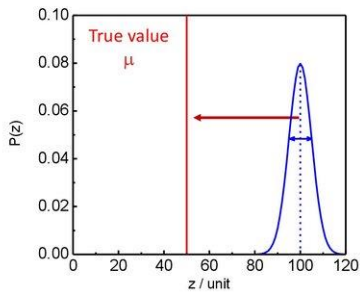
$$s_{y_j}^2 = \frac{1}{m-1} \sum_{i=1}^m (y_i - y)^2$$

Training material



$(Y_1, \dots, Y_N, B) \Rightarrow$ ESTIMATE OF $Z = Y - B$

- Z : measurand, i.e. quantity of interest, with true value μ
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$$y = \frac{1}{m} \sum_{j=1}^m y_j \quad u_y = \frac{s_{y_j}}{\sqrt{m}}$$

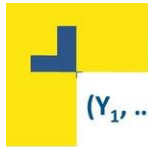
$$s_{y_j}^2 = \frac{1}{m-1} \sum_{i=1}^m (y_i - y)^2$$

$$z = y - b$$

\Rightarrow Not correcting for the background results in a systematic error b

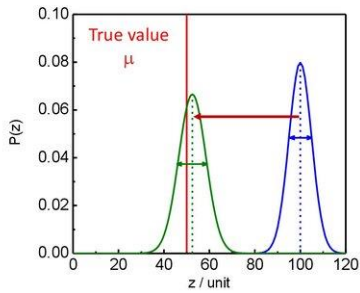
Training material





$(Y_1, \dots, Y_N, B) \Rightarrow$ ESTIMATE OF $Z = Y - B$

- Z : measurand, i.e. quantity of interest, with true value μ
- y_i : result of a single experiment to estimate true value of Z
all measurements in same conditions (e.g. measurement time,...)
- b : correction for background, $z = y - b$



$$y = \frac{1}{m} \sum_{j=1}^m y_j \quad u_y = \frac{s_{y_j}}{\sqrt{m}}$$

$$s_{y_j}^2 = \frac{1}{m-1} \sum_{i=1}^m (y_i - y)^2$$

$$z = y - b \quad u_z = \sqrt{\frac{s_{y_j}^2}{m} + u_b^2}$$

After correction for the systematic effect (e.g. background)
 \Rightarrow Additional uncertainty u_b due to a systematic effect

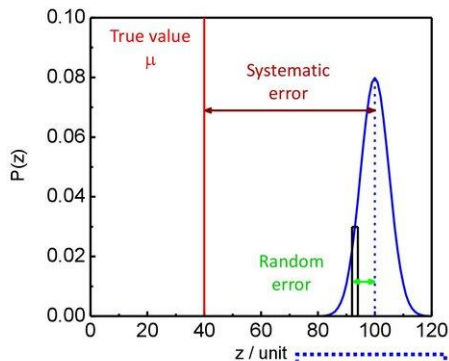


Training material



RANDOM AND SYSTEMATIC ERROR

- z_j : result of a single measurement to estimate the value of quantity with true value μ



$$z_i = \mu + \beta_s + \epsilon_{r,i}$$

- **Systematic error** : β_s

$$\beta_s = z - \mu$$

- **Random error** : $\epsilon_{r,i}$

$$\epsilon_{r,i} = z_i - z$$

$$z = \frac{1}{m} \sum_{j=1}^m z_j$$



Training material



MEASUREMENT ACCURACY AND PRECISION

- **Measurement precision**

“Closeness of agreement between indications or measured quantity values obtained by replicate measurements on the same or similar objects under specified conditions”

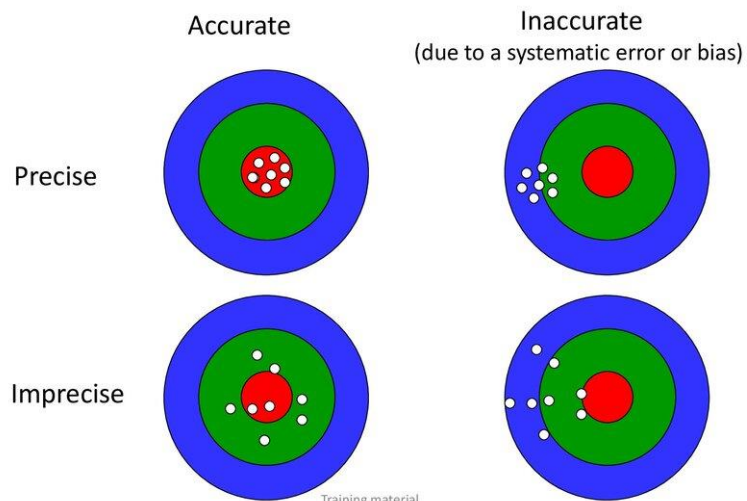
- **Measurement accuracy**

“Closeness of agreement between a measured quantity value and a true quantity value of a measurand”

Training material

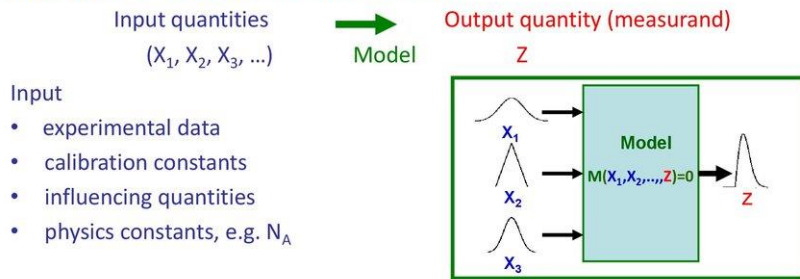


MEASUREMENT ACCURACY AND PRECISION





ASSUMPTION OF NORMAL DISTRIBUTIONS



Ideally : define Probability Distribution (PD) of (X_1, X_2, X_3, \dots) and transform into PD of Z by Monte Carlo simulations (stochastic)

Common practice: GLUP (sandwich formula based on normal distributions)

in most cases fulfilled, even if the model includes parameter adjustments

⇒ Requires a good understanding of the measurement/calculation process
(try always to start from independent input quantities)

Training material



UNCERTAINTY EVALUATION AND PROPAGATION

- Importance of the use of proper definitions and terminology
e.g. in this work all uncertainties are reported as standard deviations ($\approx 68\%$ confidence limit)
- Methods (statistical analysis) to evaluate and propagate uncertainties are well understood
- Uncertainty evaluation and propagation requires
 - Separation of random and systematic effects
 - Correlated and uncorrelated uncertainty components

⇒ Requires a good understanding of the measurement/calculation process

Training material





SNF CHARACTERISATION

An accurate characterisation of **Spent Nuclear Fuel (SNF)** is required for a **safe, ecological and cost effective** operation of the facilities involved in **SNF management** (transport, storage, handling, reprocessing and disposal):

- Decay heat
- Neutron emission
- γ -ray emission
- Reactivity (Burn Up Credit (BUC), i.e. Fission Product (FP), actinides)
- Fissile material (Nuclear Safeguards, i.e. ^{235}U , ^{239}Pu)
- Specific nuclides (Long term safety)
i.e. ^{14}C , ^{36}Cl , ^{79}Se , ^{94}Nb , ^{99}Tc , ^{129}I , ^{135}Cs , ^{226}Ra , ^{237}Np



A reliable determination of these observables including **realistic uncertainties** requires a **good understanding** of the underlying physics process

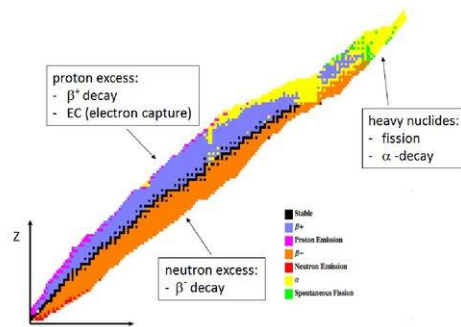
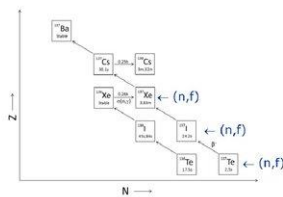


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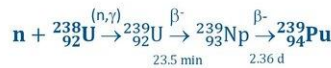


SPENT NUCLEAR FUEL: NUCLIDE INVENTORY

- Fission products (FP): neutron induced fission (n,f)
e.g. ^{90}Sr , ^{99}Tc , ^{137}Cs , ^{154}Eu , ...



- Actinides (major and minor): neutron induced capture (n, γ)
e. g. ^{235}U , ^{238}U , ^{239}Pu , ^{241}Am , ^{244}Cm , ...

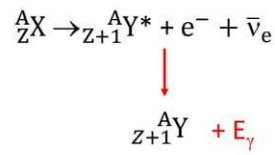
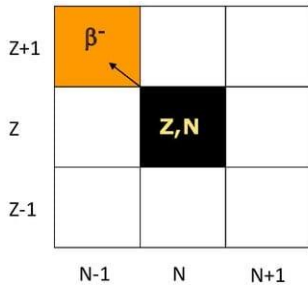


Training material



β^- - DECAY BY NEUTRON RICH NUCLIDES

Radioactive decay in which a neutron is transformed into a proton and an electron + anti-neutrino are emitted

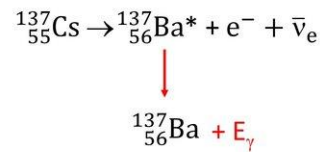
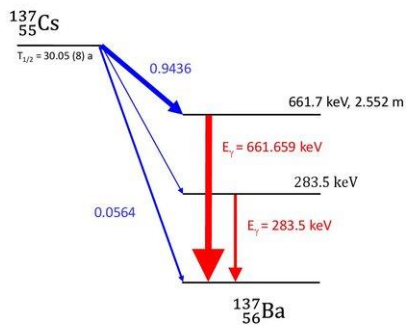


Training material



β^- - DECAY BY NEUTRON RICH NUCLIDES: e.g. ^{137}Cs

- $T_{1/2} = 30.05 (8) \text{ a}$
- $Q_{\beta^-} = 1175.73 (17) \text{ keV}$



Gamma-ray emission probability: P_γ

E_γ	P_γ
661.7 keV	0.8499 (20)
283.5 keV	$5.8 (8) \times 10^{-6}$

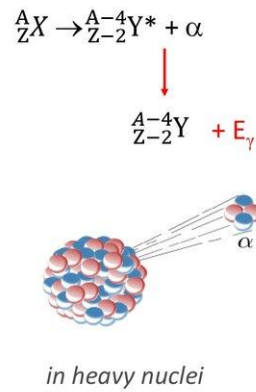
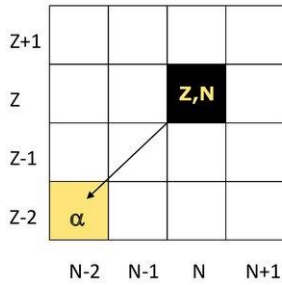
http://www.nucleide.org/DDEP_WG/Nuclides/Cs-137_tables.pdf



Training material

α - DECAY

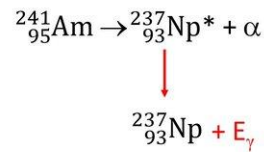
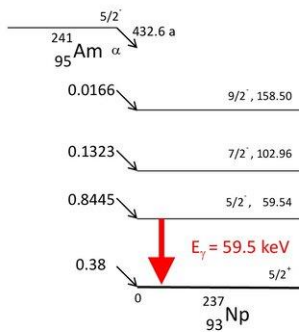
Radioactive decay in which a nucleus emits an α-particle (⁴He nucleus) and transforms into a nucleus with 4 less nucleons (2n, 2p)



Training material

α - DECAY BY ACTINIDES: e.g. ²⁴¹Am

- $T_{1/2} = 432.6 (6) \text{ a}$
- $Q_\alpha = 5637.82 (12) \text{ keV}$



α -particle emission probability:

E_α	$P(E_\alpha)$
5388.3 keV	0.0166 (3)
5442.9 keV	0.1323 (10)
5485.6 keV	0.8445 (10)

http://www.nucleide.org/DDEP_WG/Nuclides/Am-241_tables.pdf



Training material

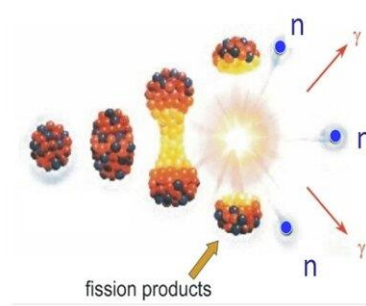


SPONTANEOUS FISSION

Fission: nucleus splits up into fission products (FP)

e.g. $^{238,240,242}\text{Pu(sf)}$, $^{242,244,246}\text{Cm(sf)}$, $^{252}\text{Cf(sf)}$

- FP : acceleration due to Coulomb repulsion
- FP : strongly excited, emission of
 - Prompt fission neutrons (PFN)
 - Prompt fission γ -rays (PFG)
- Prompt fission neutrons important for Non-Destructive Assay (NDA)



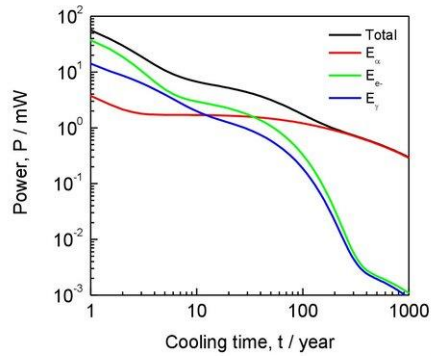
Training material



DECAY POWER PRODUCED BY SNF

- Initial enrichment (IE)
 - amount of ^{235}U relative to the total amount of U in the fresh fuel (before irradiation in the reactor)
 - $^{235}\text{U}/\text{U}$ mostly expressed in wt%
- Cooling time (CT)
 - Time period since the removal of a reactor (running)
- Burnup (BU)
 - Measure of the total energy production
 - $E_{\text{tot}} \approx E_{\text{ff}} \times N_{\text{f}}$
 - E_{tot} : total energy
 - E_{ff} : energy per fission
 - N_{f} : total number of fission event
 - Mostly defined as: total energy divided by total amount of initial fuel
 - Expressed in MWd/kg (time integrated power/amount of initial fuel)

PWR UO_2 pellet (5 g)
 $^{235}\text{U}/\text{U} = 4.8 \text{ wt\%}$
 burnup = 45 MWd/kg



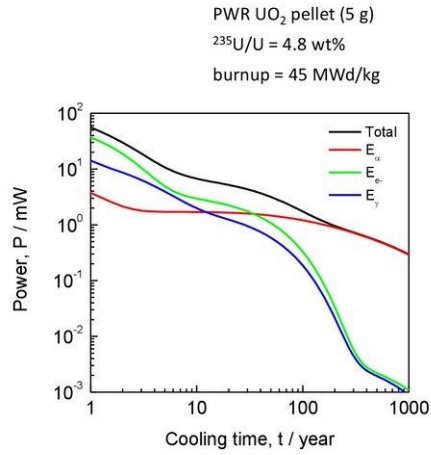
Training material



DECAY POWER PRODUCED BY SNF

$$P(t) = \sum_k P_k(t)$$

- $P_k(t)$: contribution of radionuclide k
- $P_k(t) = p_k N_k(t)$
 - $N_k(t)$: number of nuclei of nuclide k at time t
 - p_k : specific decay heat rate of nuclide k

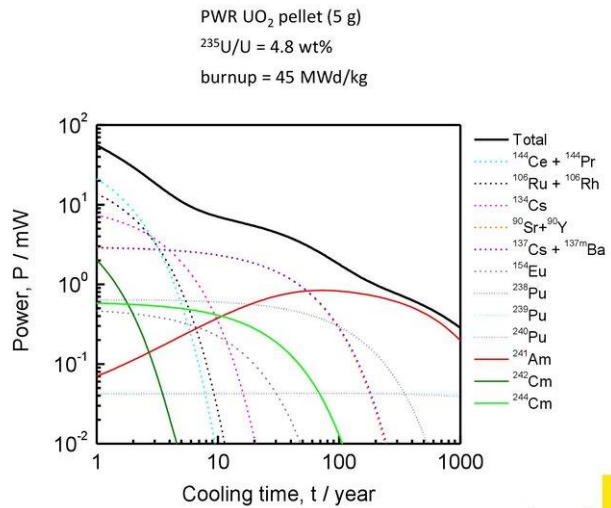


Training material



DECAY POWER PRODUCED BY SNF

- $1 \text{ a} \leq t \leq 10 \text{ a}$
 - ¹⁴⁴Ce / ¹⁴⁴Pr
 - ¹⁰⁶Ru / ¹⁰⁶Rh
 - ¹³⁴Cs
 - ⁹⁰Sr / ⁹⁰Y
 - ¹³⁷Cs / ^{137m}Ba
- $10 \text{ a} \leq t \leq 100 \text{ a}$
 - ⁹⁰Sr / ⁹⁰Y
 - ¹³⁷Cs / ^{137m}Ba
 - ²³⁸Pu
 - ²⁴¹Am
 - ²⁴⁴Cm
- $100 \text{ a} \leq t$
 - ²⁴¹Am
 - ²³⁸Pu
 - ^{239,241}Pu



Training material

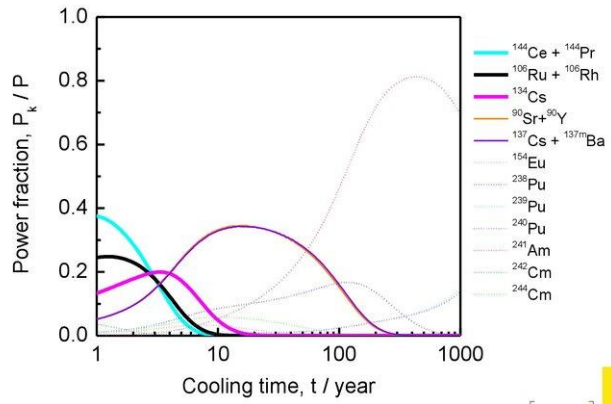




DECAY POWER PRODUCED BY SNF

- $1 \text{ a} \leq t \leq 10 \text{ a}$
 - $^{144}\text{Ce} / ^{144}\text{Pr}$
 - $^{106}\text{Ru} / ^{106}\text{Rh}$
 - ^{134}Cs
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
- $10 \text{ a} \leq t \leq 100 \text{ a}$
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
 - ^{238}Pu
 - ^{241}Am
 - ^{244}Cm
- $100 \text{ a} \leq t$
 - ^{241}Am
 - ^{238}Pu
 - $^{239,241}\text{Pu}$

PWR UO₂ pellet (5 g)
²³⁵U/U = 4.8 wt%
 burnup = 45 MWd/kg



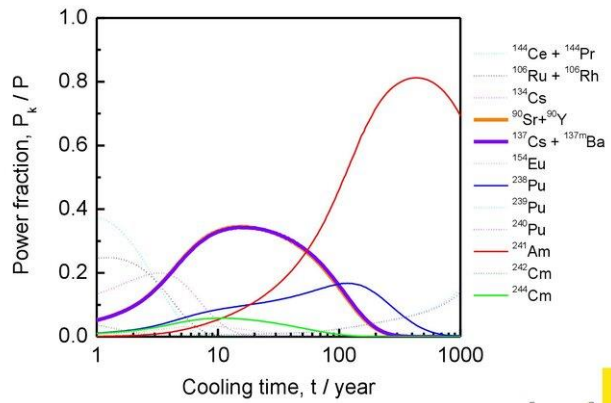
Training material



DECAY POWER PRODUCED BY SNF

- $1 \text{ a} \leq t \leq 10 \text{ a}$
 - $^{144}\text{Ce} / ^{144}\text{Pr}$
 - $^{106}\text{Ru} / ^{106}\text{Rh}$
 - ^{134}Cs
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
- $10 \text{ a} \leq t \leq 100 \text{ a}$
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
 - ^{238}Pu
 - ^{241}Am
 - ^{244}Cm
- $100 \text{ a} \leq t$
 - ^{241}Am
 - ^{238}Pu
 - $^{239,241}\text{Pu}$

PWR UO₂ pellet (5 g)
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 burnup = 45 MWd/kg

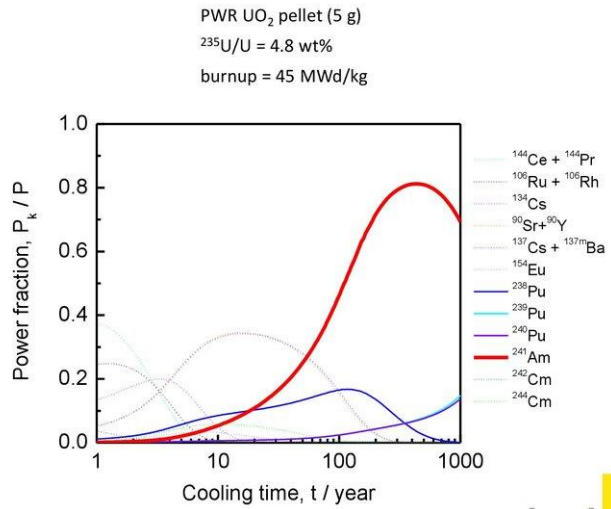


Training material



DECAY POWER PRODUCED BY SNF

- $1 \text{ a} \leq t \leq 10 \text{ a}$
 - $^{144}\text{Ce} / ^{144}\text{Pr}$
 - $^{106}\text{Ru} / ^{106}\text{Rh}$
 - ^{134}Cs
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
- $10 \text{ a} \leq t \leq 100 \text{ a}$
 - $^{90}\text{Sr} / ^{90}\text{Y}$
 - $^{137}\text{Cs} / ^{137\text{m}}\text{Ba}$
 - ^{238}Pu
 - ^{241}Am
 - ^{244}Cm
- $100 \text{ a} \leq t$
 - ^{241}Am
 - ^{238}Pu
 - $^{239,241}\text{Pu}$



Training material



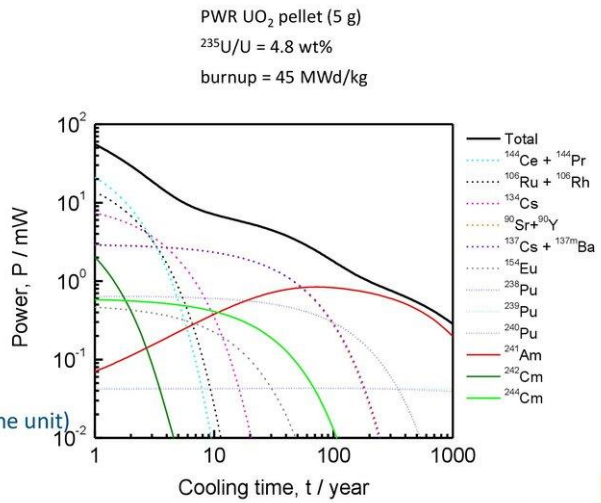
DECAY POWER PRODUCED BY SNF

$$P(t) = \sum_k p_k N_k(t)$$

- $N_k(t)$: number of nuclei at time t
 - $N_k(t_0)$: number of nuclei at time t_0
 - Nuclide vector at $t > t_0$

$$\frac{dN_k}{dt} = \sum_{i \rightarrow k} \lambda_i N_i - \lambda_k N_k$$

- $p_k = E_{dk} \lambda_k$: specific decay heat rate (nuclear data)
 - λ_k : decay constant (disintegration per time unit)
 - E_{dk} : energy per decay



Training material





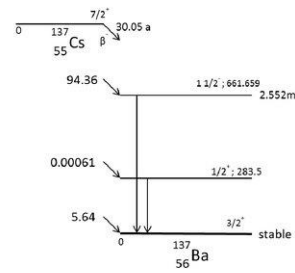
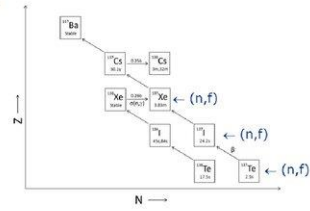
EXAMPLE: ¹³⁷Cs CONTRIBUTION TO DECAY POWER

Decay power : $P_k(t) = p_k N_k(t)$

- Production of ¹³⁷Cs:
 - (n,f) followed by β⁻ (decay of short lived precursors)
 - Small neutron capture cross section
 - ⇒ $N_k(t_0) \propto$ total number of fission reactions

- Radioactive decay: $p_k = E_{rk} \lambda_k$
 - $T_{1/2}$: 30.18 (15) a ($\lambda_k = \ln 2 / T_{1/2}$)
 - Q_{β^-} : 1175.63 (17) keV
 - E_{dk} : recoverable energy
 - Average electron + recoil energy : 247.9 (12) keV
 - $\langle E_{\gamma} \rangle$: 565.4 (13) keV

Decay data from DDEP (Decay Data Evaluation Project) : <http://www.nucleide.org/DDEP.htm>

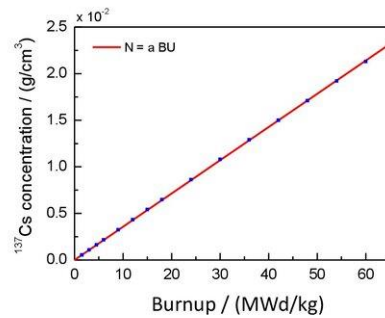


Training material



¹³⁷Cs: CONTRIBUTION TO DECAY POWER

- Total number of ¹³⁷Cs nuclei at time t_0 : $N_k(t_0) = Y_k N_f(t_0)$
 - Y_k : cumulative fission yield
 - $N_k(t_0)$: total number of fission reactions at end of reactor operation
 - Burnup, total energy production: $BU = E_{rf} N_f(t_0)$ (BU: time integrated reactor power)
 - $N_f(t_0)$: total number of fission events at end of reactor operation
 - E_{rf} : recoverable energy per fission reaction
- ⇒ Total number of ¹³⁷Cs at t_0 : $N_k(t_0) \propto BU$



Training material



¹³⁷Cs : CONTRIBUTION TO THERMAL POWER (UNCERTAINTY EVALUATION)

$$P_k(t) = p_k N_k(t_0) e^{-\lambda_k (t-t_0)} \quad N_k(t_0) \propto Y_c \text{ BU} \quad p_k = E_{dk} \lambda_k$$

$$\Rightarrow \frac{u_{P_k}}{P_k} = \sqrt{\left(\frac{u_{BU}}{BU}\right)^2 + \left(\frac{u_{Y_c}}{Y_c}\right)^2 + \left(\frac{u_{E_{dk}}}{E_{dk}}\right)^2}$$

- Nuclear data
 - Y_c : cumulative fission yields
 - E_{dk} : recoverable energy
 - λ_k : decay constant (small uncertainty)
 - Operational history : BU
- Note:
- Contribution of ¹³⁷Cs and ⁹⁰Sr: similar
 - simple production path
 - with relatively well determined nuclear data
 - Production of e.g ¹³⁴Cs more complex $\propto BU^b$ with $b \sim 2$

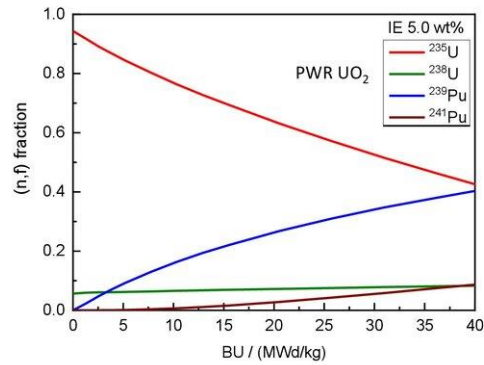
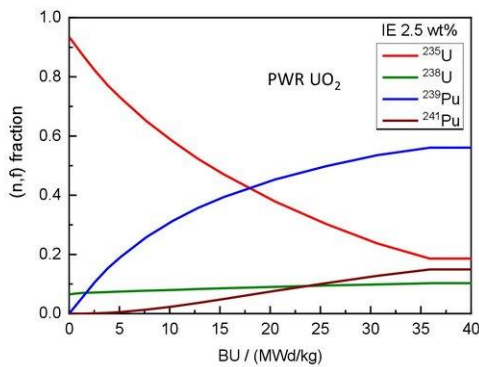
$$\Rightarrow \frac{u_{N_k}}{N_k} = 2 \frac{u_{BU}}{BU}$$



Training material



¹³⁷Cs: CONTRIBUTION OF ²³⁵U, ²³⁹Pu AND ²⁴¹Pu



Training material



¹³⁷Cs : NUCLEAR DATA (Y_c, E_d) FOR ²³⁵U(n,f) AT THERMAL ENERGY

Library	E _d / keV	Ratio	¹³⁷ Cs		(Y _c x E _d) / keV	Ratio
			100 x Y _c	Ratio		
DDEP/IAEA	811.8 (18)	1	6.221 (69)	1	50.5 (6)	1
JEF-2.2	812.0 (69)	1.000 (8)	6.244 (54)	1.004 (9)	50.7 (6)	1.004 (12)
JEFF-3.1.1	810.1 (23)	0.998 (3)	6.221 (69)	1.000 (11)	50.4 (6)	0.998 (11)
JEFF-3.3	801.8 (23)	0.988 (3)	6.090 (63)	0.979 (10)	48.8 (5)	0.967 (10)
ENDF/B-VI.8	813.4 (41)	1.002 (5)	6.188 (31)	0.995 (5)	50.3 (4)	0.997 (7)
ENDF/B-VII.0	805.7 (16)	0.992 (2)	6.188 (31)	0.995 (5)	49.9 (3)	0.987 (5)
ENDF/B-VIII.0	805.8 (18)	0.993 (2)	6.188 (31)	0.995 (5)	49.9 (3)	0.987 (5)

DDEP: <http://www.nucleide.org/DDEP.htm>
 Evaluated data libraries: https://www.oecd-nea.org/jcms/pl_39910/janis

Training material



⁹⁰Sr : NUCLEAR DATA (Y_c, E_d) FOR ²³⁵U(n,f) AT THERMAL ENERGY

Library	E _d / keV	Ratio	⁹⁰ Sr		(Y _c x E _d) / keV	Ratio
			100 x Y _c	Ratio		
DDEP/IAEA	1129.4 (14)	1	5.730 (130)	1	64.7 (15)	1
JEF-2.2	1129.6 (7)	1.000 (1)	5.847 (188)	1.020 (33)	66.0 (21)	1.021 (33)
JEFF-3.1.1	1107.8 (13)	0.981 (1)	5.729 (132)	1.000 (23)	63.5 (15)	0.981 (23)
JEFF-3.3	1127.3 (13)	0.998 (1)	5.676 (131)	0.991 (23)	64.0 (15)	0.989 (23)
ENDF/B-VI.8	1129.9 (12)	1.000 (1)	5.782 (58)	1.009 (10)	65.3 (7)	1.010 (10)
ENDF/B-VII.0	1129.4 (13)	1.000 (1)	5.782 (58)	1.009 (10)	65.3 (7)	1.009 (10)
ENDF/B-VIII.0	1128.8 (11)	0.999 (1)	5.782 (58)	1.009 (10)	65.3 (7)	1.009 (10)
Ramthum (Exp. 1967)	1147.0 (90)					

DDEP: <http://www.nucleide.org/DDEP.htm>
 Evaluated data libraries: https://www.oecd-nea.org/jcms/pl_39910/janis
 H. Ramthum, Proc. Symp. Standardization of radionuclides, Oct. (1966) (IAEA, 1967), p. 589

Training material





⁹⁰Sr and ¹³⁷Cs: NUCLEAR DATA (Y_c, E_d) FOR ²³⁵U(n,f) AT THERMAL ENERGY

Library	⁹⁰ Sr		¹³⁷ Cs	
	(E _d x Y _c) / keV	Ratio	(Y _c x E _d) / keV	Ratio
DDEP/IAEA	64.7 (15)	1	50.5 (6)	1
JEF-2.2	66.0 (21)	1.021 (33)	50.7 (7)	1.004 (12)
JEFF-3.1.1	63.5 (15)	0.981 (23)	50.4 (6)	0.998 (11)
JEFF-3.3	64.0 (15)	0.989 (23)	48.8 (5)	0.967 (10)
ENDF/B-VI.8	65.3 (7)	1.010 (10)	50.3 (4)	0.997 (7)
ENDF/B-VII.0	65.3 (7)	1.009 (10)	49.9 (3)	0.987 (5)
ENDF/B-VIII.0	65.3 (7)	1.009 (10)	49.9 (3)	0.987 (5)

EPRI, PIRT, July 2020

Table 5-5
Summary of Uncertainty in Calculated Decay Heat [17] for BWR Assembly at 37 GWd/MTU Burnup and 15.6 Years' Cooling Time

Perturbed Parameter Set	Relative Uncertainty (%)
Fuel design data	0.2
Operating history data	0.8
Nuclear cross-section data	0.9
Fission yield data	0.3
Overall uncertainty	1.3

Ilas et al., Nucl. Eng. Des. 319 (2017) 176

Data set	Data set	Uncertainty (1σ) (%)
Modeling data	Fuel design	0.20
	Operating data	0.85
	Total	0.87
Nuclear data	Cross sections	0.88
	Fission yields	0.26
	Total	0.92
Overall effect	Total	1.27

u_p/P = 1.3%, is this realistic?

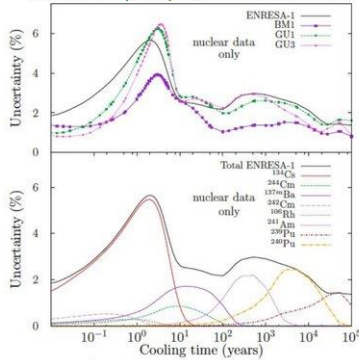


Training material



⁹⁰Sr and ¹³⁷Cs: NUCLEAR DATA (Y_c, E_d) FOR ²³⁵U(n,f) AT THERMAL ENERGY

Uncertainty only due to nuclear data > 2%



ENRESA-1: BWR
BM1 : MOX
GU1 : PWR
GU3 : PWR

EPRI, PIRT, July 2020

Table 5-5
Summary of Uncertainty in Calculated Decay Heat [17] for BWR Assembly at 37 GWd/MTU Burnup and 15.6 Years' Cooling Time

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Ilas et al., Nucl. Eng. Des. 319 (2017) 176

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	Operating data	0.85
	Total	0.87
Nuclear data	Cross sections	0.88
	Fission yields	0.26
	Total	0.92
Overall effect	Total	1.27

u_p/P = 1.3%, is this realistic? NO



Training material

EURAD WP8 Task 2

Rochman et al., Annals of Nuclear Energy 160 (2021) 108539
Rochman et al., EPJ Nuclear Sci. Technolog. 7 (2021) 18
Rochman et al., EPJ Nuclear Sci. Technolog. 8 (2022) 9

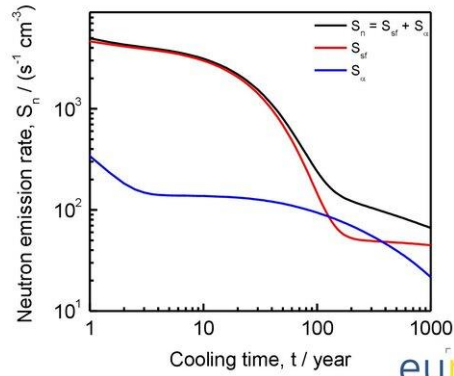


NEUTRON EMISSION BY SPENT NUCLEAR FUEL

$$S_n(t) = \sum_k S_{n,k}(t)$$

- $S_{n,k}(t)$: contribution of radionuclide k
- $S_{n,k}(t) = (s_{sf,k} + s_{\alpha,n,k}) N_k(t)$
 - $N_k(t)$: number of nuclei of nuclide k at time t
 - $s_{sf,k}$: specific neutron emission rate of nuclide k due to **spontaneous fission**
 - $s_{\alpha,k}$: specific neutron emission rate of nuclide k due to **(α ,n) reactions**

PWR UO₂ pellet (5 g)
²³⁵U/U = 4.8 wt%
 burnup = 45 MWd/kg



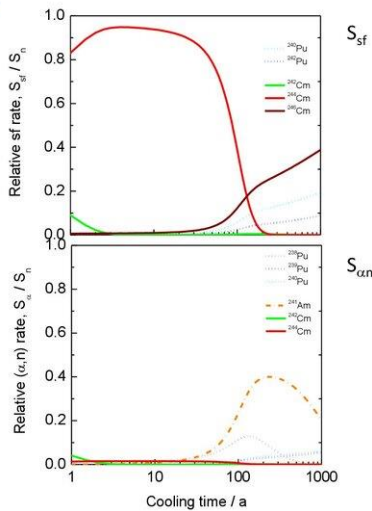
61

Training material

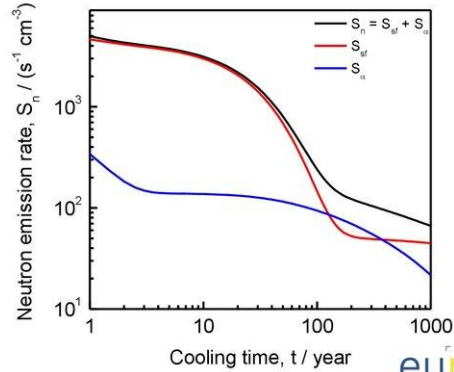


NEUTRON EMISSION BY SPENT NUCLEAR FUEL

PWR UO₂ pellet (5 g)
²³⁵U/U = 4.8 wt%
 burnup = 45 MWd/kg



⇒ Main contribution from ²⁴⁴Cm(sf)

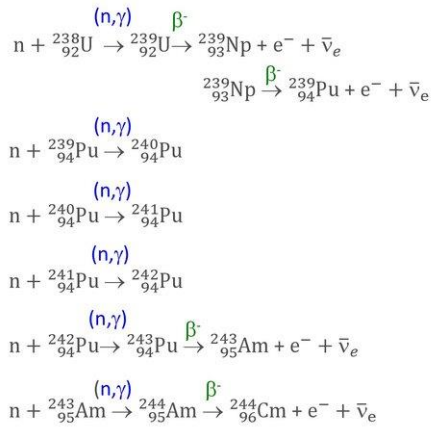


62

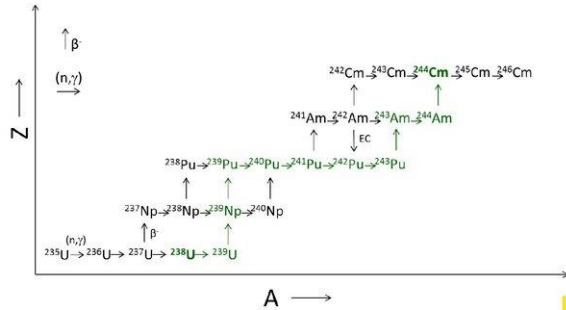
Training material



²⁴⁴Cm PRODUCTION: COMPLEX PROCESS



Sequence of
 - (n,γ) reactions (6)
 - β⁻ decays (4)



Training material



SNF SOURCE TERMS

Main **observables** or **source terms** of interest:

- Decay heat : H
- Neutron emission : S_n
- γ-ray emission : S_γ
- Reactivity : ²³⁵U, ²³⁹Pu, Fission Products (BUC)
- Fissile material : ²³⁵U, ²³⁹Pu
- Long-term safety : e.g. ¹⁴C, ³⁶Cl, ⁷⁹Se, ⁹⁴Nb, ⁹⁹Tc, ¹²⁹I, ²²⁶Ra, ²³⁷Np

difficult to be **measured** directly, in particular during **industrial operation**
can not be used to **extrapolate** to evaluate **behaviour** as function of time



Training material



SNF SOURCE TERMS: NUCLIDE INVENTORY

Nuclide	Source term	CT
⁹⁰ Sr	H, S _γ	10 a ≤ t ≤ 100 a
¹⁰⁶ Ru	H	1 a ≤ t ≤ 10 a
¹³⁴ Cs	H	1 a ≤ t ≤ 10 a
¹³⁷ Cs	H, S _γ	10 a ≤ t ≤ 100 a
¹⁴⁴ Ce	H	1 a ≤ t ≤ 10 a
¹⁴⁸ Nd	Burn-up	stable
¹⁴⁹ Sm	Power	stable
¹⁵⁴ Eu	H, S _γ	1 a ≤ t ≤ 10 a

Nuclide	Source term	CT
²³⁵ U	R, S _γ	10 ⁵ a ≤ t
²³⁸ U	R, S _γ	10 ⁵ a ≤ t
²³⁸ Pu	H, S _γ	10 a ≤ t
²³⁹ Pu	R, S _γ	100 a ≤ t ≤ 10 ⁴ a
²⁴⁰ Pu	R, S _γ	100 a ≤ t ≤ 10 ⁴ a
²⁴¹ Pu	H, S _γ	10 a ≤ t ≤ 100 a
²⁴¹ Am	H	10 a ≤ t
²⁴² Cm	H, S _n	1 a ≤ t ≤ 10 a
²⁴⁴ Cm	H, S _n	10 a ≤ t ≤ 100 a

H : thermal power or decay heat
 S_n : neutron emission
 S_γ : γ-ray emission
 R : reactivity (criticality safety)

Criticality safety (Burn Up Credit, BUC):

⁹⁵Mo, ⁹⁹Tc, ¹⁰¹Ru, ¹⁰³Rh, ¹⁰⁹Ag, ¹³³Cs, ¹⁴³Nd, ^{147,149,150,151,152}Sm, ¹⁵⁵Gd

Long term safety:

¹⁴C, ³⁶Cl, ⁷⁹Se, ⁹⁴Nb, ⁹⁹Tc, ¹²⁹I, ¹³⁵Cs, ²²⁶Ra, ²³⁷Np

⇒ Requires **complex nuclide inventory** which can only be obtained by **theoretical calculations**



Training material



DEPLETION CALCULATIONS/CODES

Coupled neutron transport – nuclide depletion/creation

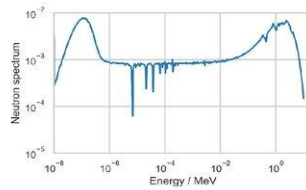
Neutron transport



Bateman equation

$$\frac{dN_k}{dt} = Y N_f \sigma_f \varphi + \sum_{i \rightarrow k} \lambda_i N_i + \sum_{j \rightarrow k} \sigma_j N_j \varphi - (\lambda_k + \sigma_{k,a} \varphi) N_k$$

Update nuclide vector



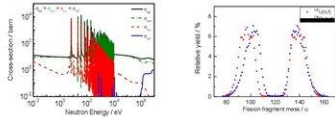
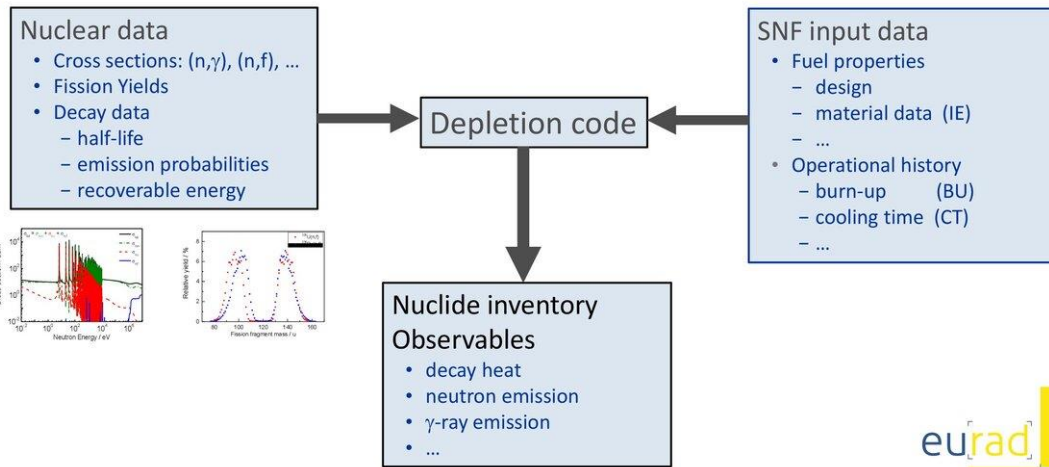
- | | |
|----------|------------|
| ALEPH2 | (SCK CEN) |
| CASMO | (STUDSVIK) |
| DARWIN | (CEA) |
| EVOLCODE | (CIEMAT) |
| SERPENT | (VTT) |
| SCALE | (ORNL) |
| STREAM | (UNIST) |
| VESTA | (IRSN) |



Training material



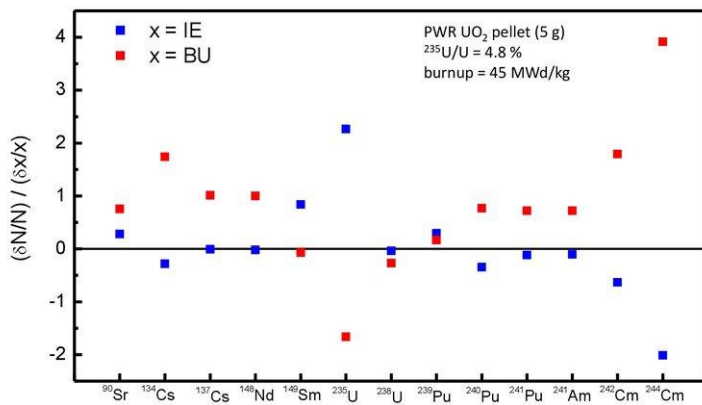
DEPLETION CODES: THEORETICAL CALCULATION OF SNF NUCLIDE INVENTORY AND OBSERVABLES



Training material



SENSITIVITY TO DESIGN AND OPERATIONAL HISTORY (DO)



Initial Enrichment (IE)
Burn-U (BU)_p

$$^{244}\text{Cm} : N = a\text{BU}^b$$

$$b \approx 4 \text{ for BU} = 45 \text{ MWd/kg}$$

$$\frac{\delta N}{N} = b \frac{\delta \text{BU}}{\text{BU}}$$

0.5 % uncertainty on BU
⇒ 2 % uncertainty on ²⁴⁴Cm inventory

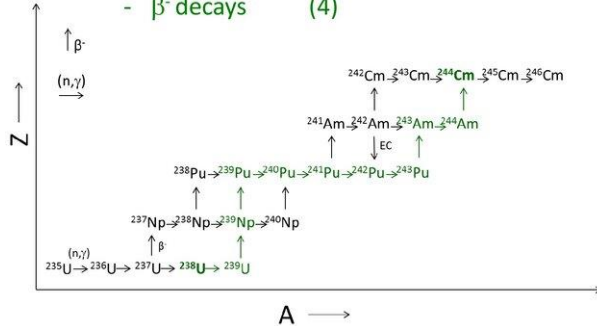
Training material





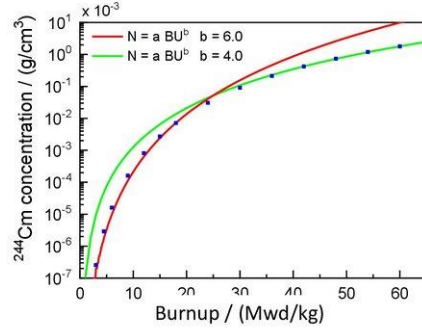
²⁴⁴Cm PRODUCTION: COMPLEX PROCESS

- Sequence of
- (n,γ) reactions (6)
 - β⁻ decays (4)



$$^{244}\text{Cm} : N = aBU^b$$

$$\frac{\delta N}{N} = b \frac{\delta BU}{BU}$$



Training material



DEPLETION CODES/CALCULATIONS

- Nuclide vector $N(t_0)$ at time t_0 after irradiation : burn-up code

Nuclear Data (ND)

- Cross sections
- Fission yields
- Neutron emission probabilities
- Decay data

Fuel History (FH)

- Fuel fabrication data (design, composition)
e.g. Initial enrichment (IE)
- Reactor operation and irradiation conditions
e.g. Burnup (BU)
- Cooling time (CT)

- Nuclide vector at $t > t_0$

$$\frac{dN_k}{dt} = \sum_{i \rightarrow k} \lambda_i N_i - \lambda_k N_k$$

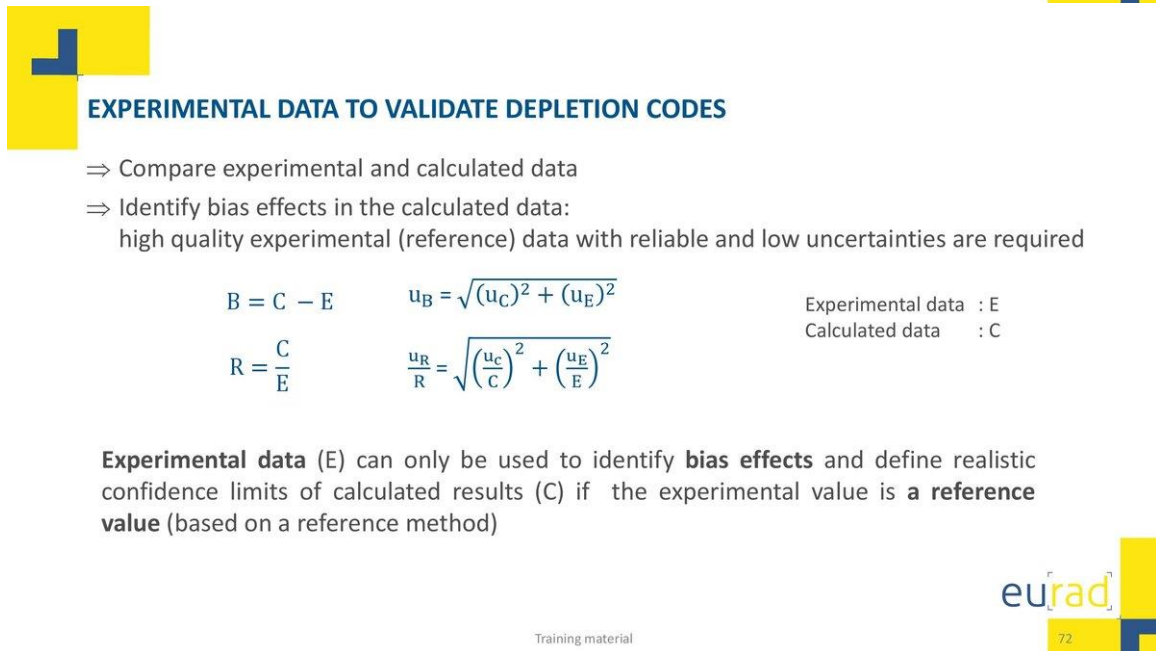
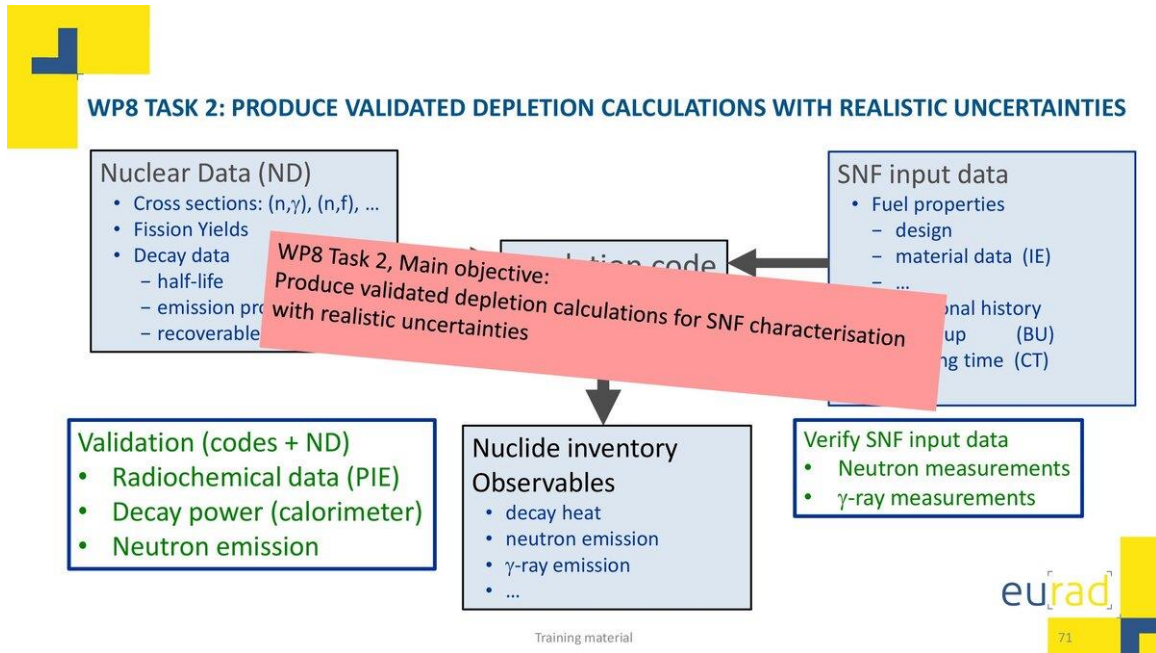
- Source terms at $t > t_0$

e.g. $P(t) = \sum_k p_k N_k(t)$

Depends on **decay data** (with relatively low uncertainties)



Training material





EXPERIMENTAL DATA TO VALIDATE DEPLETION CODES

- Results of analytical methods (destructive chemical and radiochemical analysis)
- Calorimetry data (assembly)
- Neutron emission measurements

In addition, an accurate knowledge of the design and operational history is required

Training material



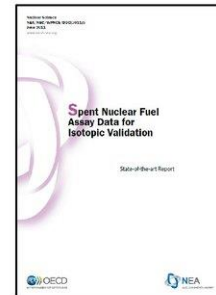
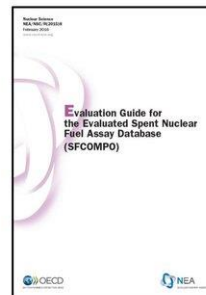
ANALYTICAL METHODS FOR SNF ASSAY DATA

Post-Irradiation Experiments (PIE)

Complex process including different steps:

- Sampling and sample dissolution
- Separation
linked to measurement technique, e.g. ICPMS)
- Measurement
 - Radiometric
(α - and γ -spectrometry, Liquid Scintillation Counting (α - and β -emitters))
 - Mass spectrometry

⇒ Data collected in SFCOMPO data base



Training material

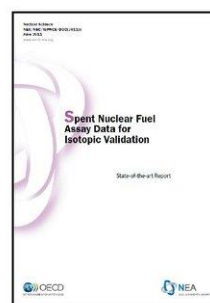




ANALYTICAL METHODS UNCERTAINTIES

Combined uncertainty of

- Sampling and sample dissolution
- Separation
- Measurement



Training material



ANALYTICAL METHOD UNCERTAINTIES: EXAMPLES

M: method % : relative uncertainties

Gauld et al., NUREG/CR-7012, January 2011

Nuclide	PNNL		JAEA		GE-VNC		ANL		Studsvik		SCK CEN		JRC KA		CEA		PSI	
	M	%	M	%	M	%	M	%	M	%	M	%	M	%	M	%	M	%
⁹⁰ Sr	LSC	5.7									LSC	8.0	ICPMS	0.4	TIMS	3.1	ICPMS	3.1
¹³⁷ Cs	γ-sp	3.5	γ-sp	3.0	γ-sp	1.75	γ-sp	4.8	γ-sp	6.4	γ-sp	2.5	ICPMS	1.5	TIMS	3.2	ICPMS	1.9
¹⁴⁸ Nd			IDMS	0.1	TIMS	0.75	ICPMS	7.1	ICPMS	4.5	TIMS	0.3	ICPMS	6.7	TIMS	3.0	ICPMS	5.0
²³⁵ U	IDMS	1.6	IDMS	0.1	TIMS	0.5	ICPMS	3.7	ICPMS	8.7	TIMS	1.0	TIMS	1.2	TIMS	3.0	ICPMS	1.9
²³⁹ Pu	IDMS	1.6	IDMS	0.3	TIMS	0.6	ICPMS	5.7	ICPMS	5.8	TIMS	0.3	TIMS	0.3	TIMS	3.1	ICPMS	4.5
²⁴⁴ Cm			MS, α	2.0	TIMS, α	2.75			ICPMS	10.0	α-sp	1.8	ICPMS	6.4	TIMS	3.1	ICPMS	1.5

Methods

- LSC : Liquid scintillation counting(β)
- α : α-spectrometry
- γ-sp : γ-spectrometry
- IDMS : Isotope dilution mass spectrometry
- ICPMS : Inductively coupled plasma mass spectrometry
- MS : mass spectrometry
- TIMS : Thermal ionisation mass spectrometry

Laboratories

- PNNL
- JAEA
- ANL:
- Studsvik
- JRC Karlsruhe
- CEA
- PSI



Training material



ANALYTICAL METHOD UNCERTAINTIES: EXAMPLES

M: method % : relative uncertainties

Gauld et al., NUREG/CR-7012, January 2011

Nuclide	Min	Max
⁹⁰ Sr	0.4 %	8.0 %
¹³⁷ Cs	1.5 %	6.4 %
¹⁴⁸ Nd	0.1 %	7.1 %
²³⁵ U	0.1 %	8.7 %
²³⁹ Pu	0.3 %	5.8 %
²⁴⁴ Cm	1.5 %	10.0 %

Uncertainty strongly depends on method and laboratory

Methods

- LSC : Liquid scintillation counting(β)
- α : α -spectrometry
- γ -sp : γ -spectrometry
- IDMS : Isotope dilution mass spectrometry
- ICPMS : Inductively coupled plasma mass spectrometry
- MS : mass spectrometry
- TIMS : Thermal ionisation mass spectrometry

Laboratories

- PNNL
- JAEA
- ANL:
- Studsvik
- JRC Karlsruhe
- CEA
- PSI



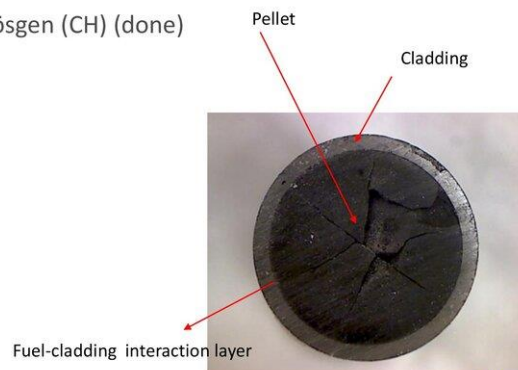
Training material



CLADDING INVENTORY

Cladding composition: important to study fuel integrity (PhD Tobias Köning, August 2022)

- Selection of Zircaloy samples irradiated in a PWR Gösgen (CH) (done)
 - Zircaloy-4 of UO_x fuel rod segment
 - Zircaloy-4 plenum of UO_x fuel rod segment
 - UO_x fuel fragment
- Experiments using various techniques at KIT
 - Scanning Electron Microscopy (SEM)
 - Wave Length X-ray spectroscopy (WDX)
 - Energy Dispersive X-ray spectroscopy (EDX)
 - X-ray Photo-electron spectroscopy (XPS)
 - Radiochemical analysis (α -spec, γ -spec, LSC, HR-ICP-MS)



Training material

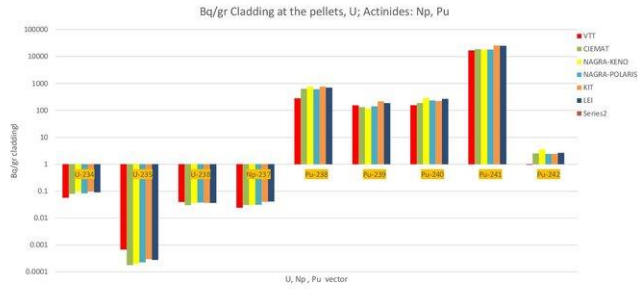


CLADDING INVENTORY

Code validation

- Calculations

- MCNP/EVOLCODE CIEMAT,
- MCNP/CINDER KIT
- SCALE (TRITON) LEI
- SCALE (KENO) NAGRA
- SCALE (POLARIS 2D) NAGRA
- VTT SERPENT2



- Interpretation of data (in progress)

- Good agreement for the cladding of the plenum and the cladding around the pellets
- Singular deviations are being checked for their origin (input issues, code deficiencies, source of uncertainties, ...)
- Comparison with experimental data used to assess how much fuel is adherent to the inner surface of the cladding during manufacturing: important to estimate the risk of radiation damage during interim storage

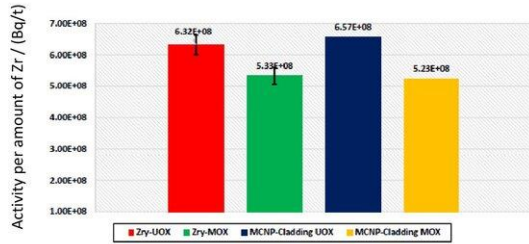


Training material



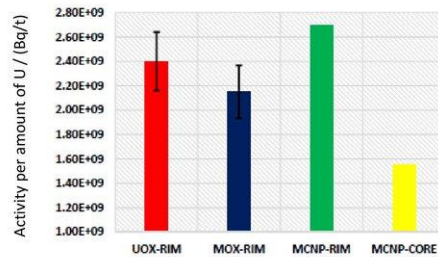
INVENTORY OF ³⁶Cl AND ¹²⁹I

³⁶Cl activity in Zry-UOx, Zry – MOX
Experimental and calculated (MCNP/CINDER)



(PhD Tobias Köning, August 2022)

¹²⁹I activity (inventory)
Experimental and calculated (MCNP/CINDER)



- First ever analysis of ³⁶Cl in PWR fuel rod components
- Calculations: 15 ppm ³⁵Cl in fuel and zircaloy (Häkkinen, Impurities in LWR fuel and structural materials, Finland 2019)

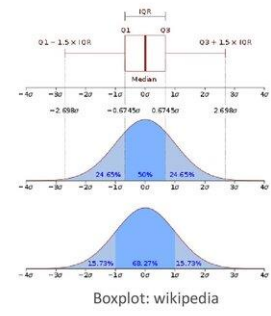
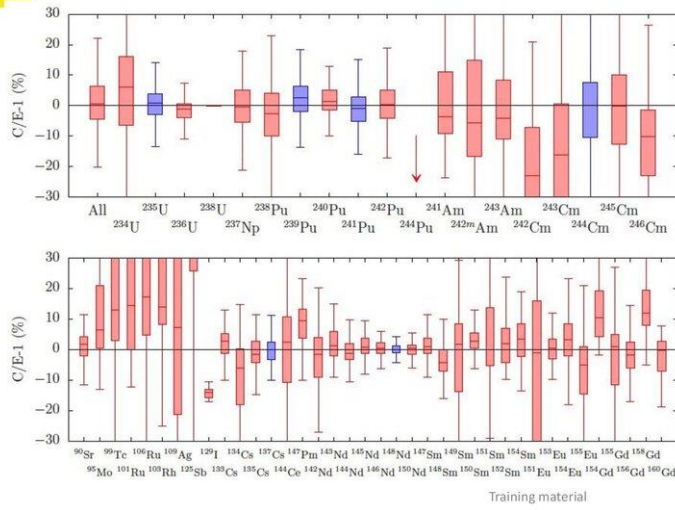
[HÄK19] S. Häkkinen, *Impurities in LWR fuel and structural materials*, VTT, Finland (2019).



Training material



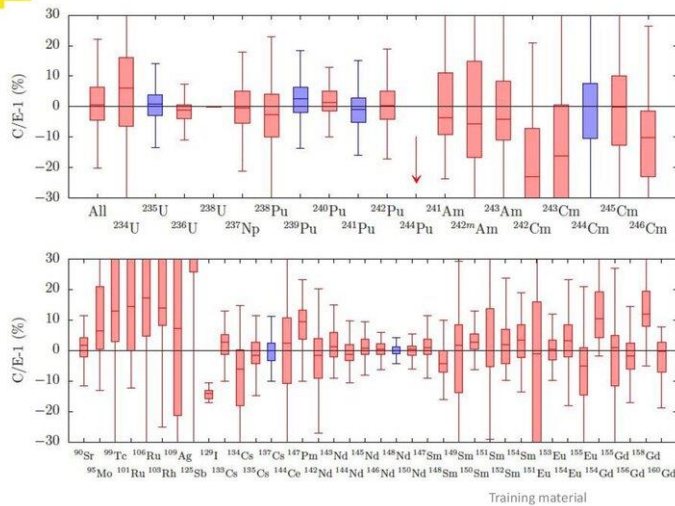
RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA



EURAD
Rochman et al., accepted in EPJ Nuclear Sci. Technol.



RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA



- C : calculated inventory
 - E : experimental inventory
- (without any selection criterion)

	<C/E>	Std. x 100
⁹⁰ Sr	0.992	8.2
¹³⁷ Cs	0.996	5.2
¹⁴⁸ Nd	0.999	2.9
²³⁵ U	1.002	7.9
²³⁹ Pu	1.025	7.9
²⁴⁴ Cm	1.020	24.0

EURAD
Rochman et al., accepted in EPJ Nuclear Sci. Technol.





RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA

Most of the data are normalised to the ^{148}Nd inventory

- C : calculated inventory
 - E : experimental inventory
- (without any selection criterion)

	<C/E>	Std. x 100
^{90}Sr	0.992	8.2
^{137}Cs	0.996	5.2
^{148}Nd	0.999	2.9
^{235}U	1.002	7.9
^{239}Pu	1.025	7.9
^{244}Cm	1.020	24.0

EURAD
Rochman et al., accepted in EPJ Nuclear Sci. Technol.



Training material



RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA

Data without any selection criterion

- Experimental data that can be biased and have large uncertainties have the same contribution compared to accurate data with low uncertainties
- Presence of dependent calculated data based on e.g. the same
 - Nuclear data library
 - Model assumptions
 - ...

- C : calculated inventory
 - E : experimental inventory
- (without any selection criterion)

	<C/E>	Std. x 100
^{90}Sr	0.992	8.2
^{137}Cs	0.996	5.2
^{148}Nd	0.999	2.9
^{235}U	1.002	7.9
^{239}Pu	1.025	7.9
^{244}Cm	1.020	24.0

EURAD
Rochman et al., accepted in EPJ Nuclear Sci. Technol.



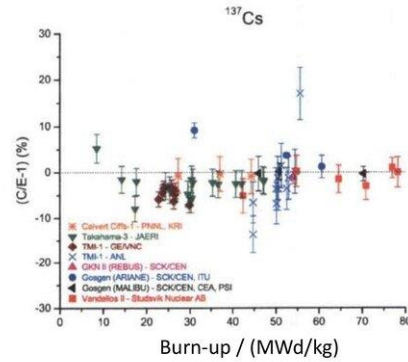
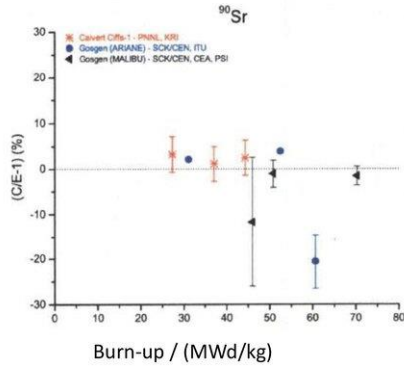
Training material



DETAILED STUDY ORNL

Gauld et al., NUREG/CR-7012, January 2011

- Selection of high quality experimental data
- One calculation code and library:
SCALE 5.1 (TRITON/NEW) & ENDF/B-V



Training material



RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA

	<C/E>	Std. x 100
90Sr	0.975	8.4
137Cs	0.978	4.4
148Nd		
235U	1.015	6.4
239Pu	1.058	6.1
244Cm	0.976	11.1

Gauld et al., NUREG/CR-7012
SCALE 5.1, ENDF/B-V

	<C/E>	Std. x 100
	0.992	8.2
	0.996	5.2
	0.999	2.9
	1.002	7.9
	1.025	7.9
	1.020	24.0

EURAD
Rochman et al.,
accepted in EPJ Nuclear Sci. Technol.

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RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA

	<C/E>	Std. x 100	<C/E>	Std. x 100	<C/E>	Std. x 100
⁹⁰ Sr	0.975	8.4	0.991	6.9	0.992	8.2
¹³⁷ Cs	0.978	4.4	0.993	3.1	0.996	5.2
¹⁴⁸ Nd			1.006	1.4	0.999	2.9
²³⁵ U	1.015	6.4	1.012	3.5	1.002	7.9
²³⁹ Pu	1.058	6.1	1.041	3.5	1.025	7.9
²⁴⁴ Cm	0.976	11.1	0.956	11.1	1.020	24.0

Gauld et al., NUREG/CR-7012
SCALE 5.1, ENDF/B-V

G. Ilas, ORNL/SPR-2019/1143
SCALE 6.1, ENDF/B-VII

EURAD
Rochman et al.,
accepted in EPJ Nuclear Sci. Technol.



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RADIOCHEMICAL ANALYSIS: ANALYSIS OF PUBLISHED DATA

	<C/E>	Std. x 100	<C/E>	Std. x 100
⁹⁰ Sr	0.975	8.4	0.991	6.9
¹³⁷ Cs	0.978	4.4	0.993	3.1
¹⁴⁸ Nd			1.006	1.4
²³⁵ U	1.015	6.4	1.012	3.5
²³⁹ Pu	1.058	6.1	1.041	3.5
²⁴⁴ Cm	0.976	11.1	0.956	11.1

Gauld et al., NUREG/CR-7012
SCALE 5.1, ENDF/B-V

G. Ilas, ORNL/SPR-2019/1143
SCALE 6.1, ENDF/B-VII

Results not consistent with

EPRI, PIRT, July 2020

Table 5-5
Summary of Uncertainty in Calculated Decay Heat [17] for BWR Assembly at 37
GWd/MTU Burnup and 15.6 Years' Cooling Time

Perturbed Parameter Set	Relative Uncertainty (%)
Fuel design data	0.2
Operating history data	0.8
Nuclear cross-section data	0.9
Fission yield data	0.3
Overall uncertainty	1.3

Ilas et al., Nucl. Eng. Des. 319 (2017) 176

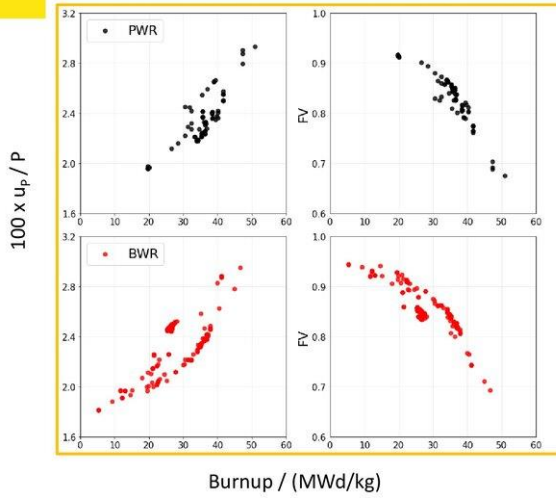
Data set	Data set	Uncertainty (1 σ) (%)
Modeling data	Fuel design	0.20
	Operating data	0.85
	Total	0.87
Nuclear data	Cross sections	0.88
	Fission yields	0.26
	Total	0.92
Overall effect	Total	1.27



Training material



CALCULATED DECAY POWER: UNCERTAINTY EVALUATION



EURAD
Shama et al., Nucl Eng. And Technol. 53 (2021) 2816

- P : calculated decay power
- u_p : total uncertainty due to
 - Nuclear data (ND)
 - Design and operational history (DO)

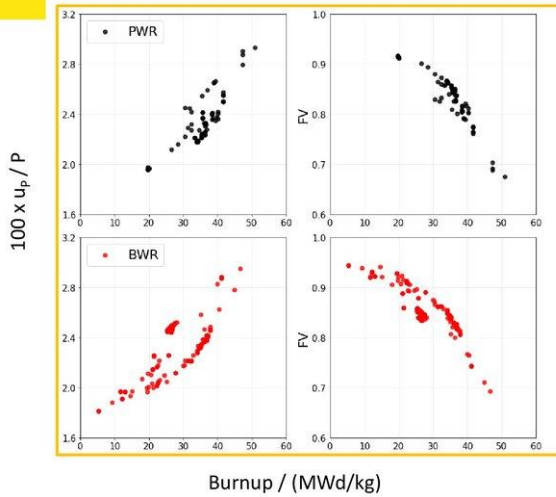
FV : fraction of variance due to DO



Training material



CALCULATED DECAY POWER: UNCERTAINTY EVALUATION



EURAD
Shama et al., Nucl Eng. And Technol. 53 (2021) 2816

- Design and operational history
- NEA/NSC/R(2015)8
- Evaluation Guide for SFCOMPO
 - **BU : 1.7 %**
 - Others see table

Table 4
Uncertainties in DO variables of the analyzed SFAs, based on [24].

Parameter *	Uncertainty/tolerance [24]	1 σ (this work)
Cladding/tube thickness	±40-50 μm	18.7 μm [†]
Cladding/tube diameter	±200 μm (PWR)/±300 μm (BWR)	67 μm/100 μm [†]
Fuel pellet density	<2% the theoretical density	0.67% [†]
Fuel pellet diameter	±30 μm	†
Enrichment (U-235 wt%)	±0.05%	0.0167% [†]
SFA powers	-	1.67%
Water temp. (PWR only)	±2 °C	2 °C
Water density (PWR only)	±0.005 g/cm ³	0.005 g/cm ³
Void fraction (BWR only)	±0%	6%
Fuel temp.	±50 °C	50 °C
Boron content (PWR only)	±10 ppm	10 ppm



Training material

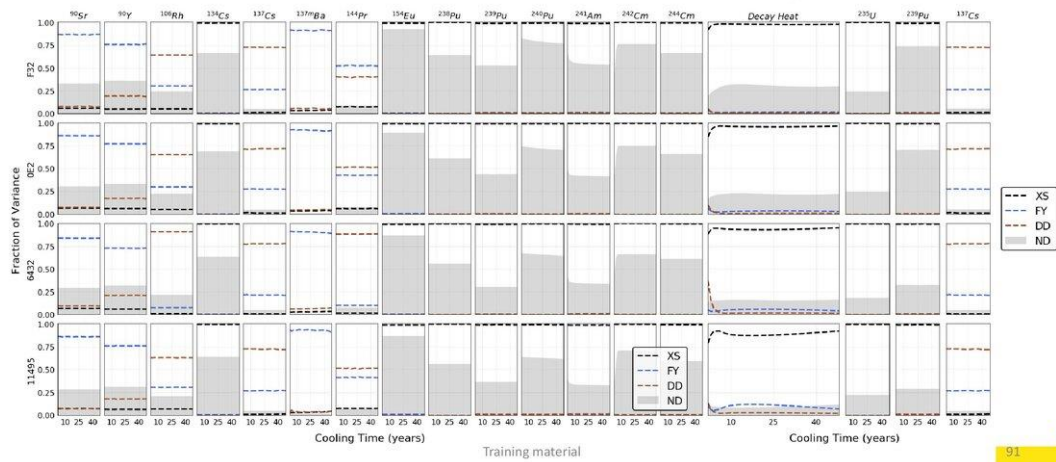


CALCULATED DECAY POWER: UNCERTAINTY EVALUATION

Contribution of XS, FY, DD, also contribution of ND and DO parameters

EURAD

Shama et al., Nucl Eng. And Technol. 53 (2021) 2816



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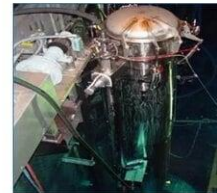
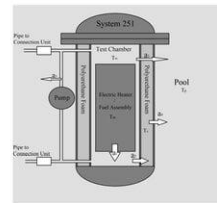


VALIDATION OF CALCULATIONS: CLAB CALORIMETER

- Only operating calorimeter worldwide to determine decay power of SNF assemblies
- Extensively used to validate depletion codes, e.g.
 - Ilas and Burns, Nucl. Tech. 208 (2022) 403
 - Shama et al., Ann. Nucl. Energy 165 (2022) 108758
 - Ebiwonjumi et al., Ann. Nucl. Energy 124 (2019) 80
 - Yamamoto and Iwahashi J. Nucl. Sci. Technol. 53 (2016) 2108
 - Haack and Ichou, EPJ Web of Conferences 146 (2017) 06012
 - San-Felice et al., Nucl. Technol. 184 (2013) 217

SCALE 6.2.4
ORIGE, CASMOS
STREAM
ORIGEN2.2, CASMOS
VESTA
DARWIN

Summary in Rochman et al., EPJ Nucl. Sci. Technol. accepted for publication



Training material

eurad

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DATA FROM CLAB CALORIMETER: SOME RESULTS

Ref.	Year	Code	Library	PWR		BWR	
				<C/E>	St. dev	<C/E>	St. dev
Ilas and Gauld	2008	SCALE 5.1	ENDF/B-V	1.011	0.012	1.003	0.025
Ilas et al.	2014	SCALE 6.1.2	ENDF/B-VII.0	1.002	0.012	0.997	0.024
Ilas and Burns	2021	SCALE 6.2.4	ENDF/B-VII.0	1.013	0.013	1.002	0.012
		SCALE 6.2.4	ENDF/B-VII.1	1.008	0.012	1.009	0.024
Ilas and Burns	2022	SCALE 6.3	ENDF/B-VIII.0	1.006	0.014	1.007	0.024
Shama et al.	2022	SCALE 6.2.3	ENDF/B-VII.1	1.019	0.012	1.003	0.025
		SCALE 6.2.3 (POLARIS)	ENDF/B-VII.1	1.015	0.012	1.010	0.026
Shama et al.	2022	CASMO5		1.009	0.013	1.008	0.025
Yamamoto	2016	CASMO5	JENDL-4.0	1.016	0.013	1.001	0.024
Haeck et al.	2022	VESTA 2.1	JEFF-3.1	0.978	0.012		
		VESTA 2.1	ENDF/B-VII.0	0.996	0.012		
San-Felice et al.	2012	DARWIN	JEFF-3.1.1	0.978	0.011		

Not blind!
Normalisation not specified
Adjustments not excluded

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BLIND TEST CLAB CALORIMETER

NUCLEAR SCIENCE AND ENGINEERING - VOLUME 196 - 1125-1145 - SEPTEMBER 2022
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DOI: <https://doi.org/10.1080/00295639.2022.2053489>



Blind Benchmark Exercise for Spent Nuclear Fuel Decay Heat

- PWR assemblies
- Decay heat rate determined at CLAB



Assembly ID	BU	CT	IE	Decay heat rate	Gamma-escape
BT01	53 MWd/kg	4.5 a	3.95 wt%	1662 W	58 W
BT02	55 MWd/kg	8.6 a	3.95 wt%	1068 W	30 W
BT03	50 MWd/kg	9.8 a	3.95 wt%	895 W	21 W
BT04	51 MWd/kg	13.5 a	3.70 wt%	768 W	15 W
BT05	50 MWd/kg	21.4 a	3.60 wt%	663 W	12 W

Training material





BLIND TEST CLAB CALORIMETER: CODES + LIBRARIES

Code	Library	Appendix Section
ALEPH 2.7.2	ENDF/B-VII.1	A.IA
APOLLO2.8/DARWIN2.3	JEFF-3.1.1	A.IB
CASMO-4E + ORIGEN-S	JEFF-2.2	A.IC
CASMO-5 (2.03)	ENDF/B-VII.1	A.ID
CASMO-5 (2.12.00) + SNF (1.07.02)	ENDF/B-VII.1	A.IE
DRAGON 4.0.5	ENDF/B-VII.1	A.IF
EVOLCODE (MCNP + ACAB)	JEFF-3.3	A.IG
MCNP-CINDER + Nukleonika (2D)	ENDF/B-VII.1	A.IH
Monteburns v3 + CINDER	ENDF/B-VII.1	A.II
MOTIVE (KENO-VI + VENTINA)	ENDF/B-VII.1	A.IJ
MOTIVE (OpenMC + VENTINA)	ENDF/B-VIII	A.IK
MVP 3	ENDF/B-VII.1	A.IL
MVP 3	JEFF-3.2	A.IM
MVP 3	JENDL-4.0	A.IN
OREST	JEFF-2.2 + ENDF/B-VI	A.IO
SCALE 6.0: ORIGEN-ARP	ENDF/B-V	A.IP
SCALE 6.1.3: ORIGEN-ARP	ENDF/B-V	A.IQ
SCALE 6.2.3: ORIGAMI	ENDF/B-VII.1	A.IP
SCALE 6.2.3: Polaris	ENDF/B-VII.1	A.IR
SCALE 6.2.3: ORIGEN	ENDF/B-VII.1	A.IR
SCALE 6.2.3: TRITON/KENO	ENDF/B-VII.1	A.IS
SCALE 6.2.3: TRITON/NEWT	ENDF/B-VII.1	A.IT
SEADep	JEFF-3.1.1	A.IU
Serpent 2.1.29	ENDF/B-VII.1	A.IV
Serpent 2.1.29	JEFF-3.1.1	A.IW
Serpent 2.1.31	JEFF-3.2 + JEFF-3.1.1	A.IX

Training material



BLIND TEST CLAB CALORIMETER: RESULTS

Assembly ID	P	CT	<C/E>	St. dev
BT01	1662 W	4.5 a	0.975	0.019
BT02	1068 W	8.6 a	0.977	0.018
BT03 (+ Gd)	895 W	9.8 a	0.967	0.019
BT04	768 W	13.5 a	0.994	0.023
BT05	663 W	21.4 a	0.979	0.021



Training material





DATA FROM CLAB CALORIMETER (R-05-62)

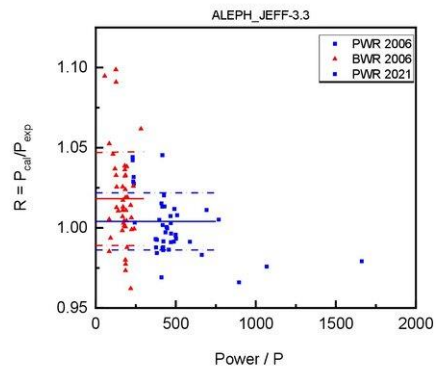
EURAD Task 2: systematic study of published calorimeter data

Compare different codes/libraries

- Total number of fissions
- Inventory of key nuclides
- Calculate decay power using recommended decay data

Paper on preparation
Romojaro et al., SCK CEN, IRSN, JSI, JRC, PSI, NAGRA

Code	Library	PWR		BWR	
		<C/E>	St. dev	<C/E>	St. dev
SCALE	(R-05-62)	1.011	0.012	1.003	0.025
ALEPH2	JEFF-3.1.2	1.001	0.020	1.013	0.031
ALEPH2	JEFF-3.3	1.004	0.018	1.018	0.029
ALEPH2	JEFF-4T0	1.006	0.018	1.020	0.030



EURAD Task 2:
additional blind test exercise using SKB-50 data



Training material



UNCERTAINTY OF CALORIMETRY DATA

- Blind test paper (Jansson et al., Nucl. Sci. Eng. 196 (2022) 1125)
5% overall uncertainty
- SKB Document R-05-62 (also used in EPRI, PRIT report)
 - PWR
 - 250 W 1.8 %
 - 900 W 1.0 %
 - BWR
 - 50 W 4.2 %
 - 350 W 1.0 %

Only total (combined uncertainties) are given

No separation between

- Systematic and random effects
- Correlated and uncorrelated uncertainty components

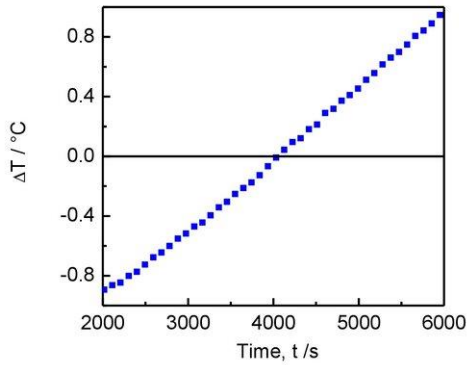
⇒ Performance assessment and uncertainty evaluation of CLAB calorimeter is required
⇒ One of the main EURAD objectives of WP8 Task 2



Training material

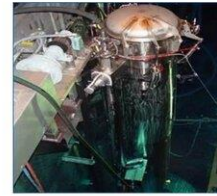
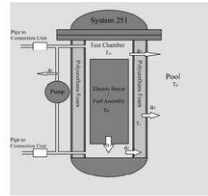


CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES



Experimental observable

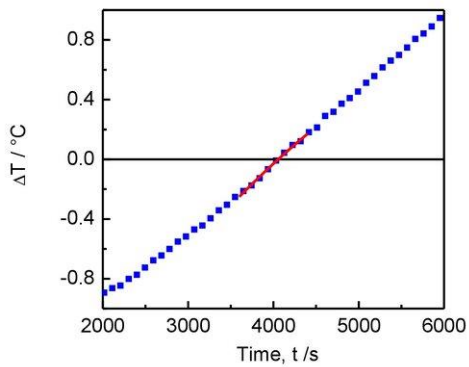
- Determine $\Delta T = T_c - T_p$ vs time



Training material



CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES



Experimental observable

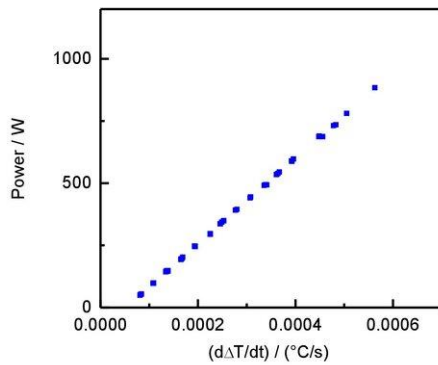
- Determine $\Delta T = T_c - T_p$ vs time
- Determine $d\Delta T/dt$ for $\Delta T = 0$



Training material



CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES



Calibration with an electrical heater

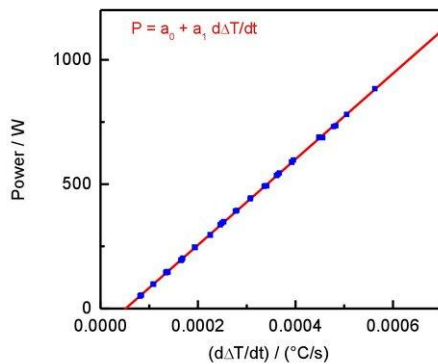
- Determine $\Delta T = T_c - T_p$ vs time
- Determine $d\Delta T/dt$ for $\Delta T = 0$
- Fit to data: $P = a_0 + a_1 d\Delta T/dt$



Training material



CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES



Calibration with an electrical heater

- Determine $\Delta T = T_c - T_p$ vs time
- Determine $d\Delta T/dt$ for $\Delta T = 0$
- Fit to data: $P = a_0 + a_1 d\Delta T/dt$



Training material



CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES

Spent nuclear fuel assembly

- Determine $\Delta T = T_c - T_p$ vs time
- Determine $d\Delta T/dt$ when $\Delta T = 0$
- $Q = a_0 + a_1 K d\Delta T/dt$
- $P = Q + P_e$
 - K : correction factor due to thermal capacity difference between electrical heater and fuel assembly
 - P_e : heat loss due to γ -rays escaping from the calorimeter

Calibration with an electrical heater

- Determine $\Delta T = T_c - T_p$ vs time
- Determine $d\Delta T/dt$ for $\Delta T = 0$
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Training material



CALORIMETER AT CLAB: DECAY POWER OF FUEL ASSEMBLIES

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Result in common uncertainty components for all assemblies measured at CLAB

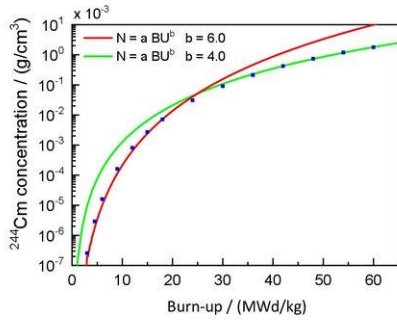
Training material





IMPORTANCE OF NEUTRON EMISSION ESTIMATION FOR SNF MANAGEMENT

$^{244}\text{Cm} : N = a\text{BU}^b$
 $b \approx 4$ for $\text{BU} = 45 \text{ MWd/kg}$ $\frac{\delta N}{N} = b \frac{\delta \text{BU}}{\text{BU}}$



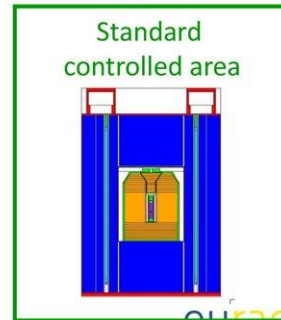
- Use neutron emission as a BU indicator to improve the operational input for depletion calculations
- For nuclear safeguards authorities using neutron emission as a signature for the fissile material
 - At present : FORK type detectors
 - Innovative detectors being developed for disposal sites in Finland and Sweden, i.e.
 - PNAR (Passive Neutron Albedo Reactivity)
 - DDSI (Differential Die Away Self-Interrogation)
- Neutron dose

Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE

EURAD TASK 2:
NDA method to determine the neutron output of a **SNF sample** under **standard controlled area** conditions



Training material

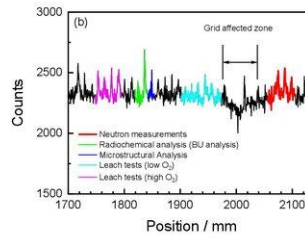




NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE

- Irradiation history burnup (REGAL project)

BU indicator	Analysis date	Nuclide Inventory N _x /N _u	BU MWd/kg
¹³⁷ Cs	21/10/2013	2.539 (55) x 10 ⁻³	52.6 (11)
¹⁴³⁺¹⁴⁴ Nd	05/02/2014	5.701 (60) x 10 ⁻³	53.95 (56)
¹⁴⁵⁺¹⁴⁶ Nd	05/02/2014	3.643 (38) x 10 ⁻³	53.05 (56)
¹⁴⁸ Nd	05/02/2014	0.974 (21) x 10 ⁻³	53.3 (12)
¹⁵⁰ Nd	05/02/2014	0.463 (21) x 10 ⁻³	52.2 (23)
Average: 52.78 (37)			



- SNF segment sample: characteristics

Parameter	Value
Segment length	52.01 (4) mm
Segment weight	42.616 (1) g
Cladding weight	6.71 (4) g
Net fuel weight	35.91 (4) g

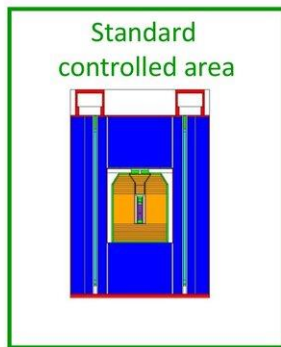
⇒ Accurate information about design and operational history



Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE



$$S_{sf} = 678 (12) \text{ s}^{-1} \text{ g}^{-1}$$

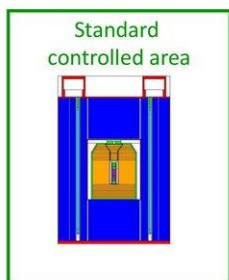
$$S_{\alpha n} / S_{sf} = 0.039 (18)$$



Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE



- ⇒ Improve
- detector characteristics
 - nuclear data

$$S_{sf} = 678 (12) \text{ s}^{-1} \text{ g}^{-1}$$

$$S_{\alpha n} / S_{sf} = 0.039 (18)$$

Uncertainty evaluation and sensitivity analysis

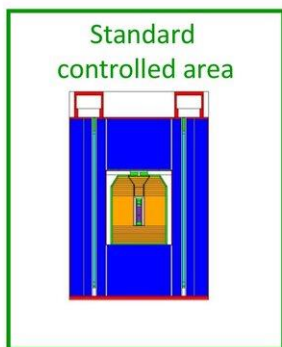
Uncertainty component, x_j	$\frac{u_{x_j}}{x_j}$	$\frac{u_{S_{sf,j}}}{u_{S_{sf}}}$	$\frac{u_{\alpha_j}}{u_{\alpha}}$
Totals rate, T	0.0008	< 0.01	0.05
Reals rate, R	0.0027	0.15	0.17
Detection efficiency, ϵ_{sf}	0.0055	0.60	0.35
Gate fraction, f	0.0071	0.40	0.45
First order factorial moment, $v_{sf(1)}$	0.0037	0.20	0.25
Second order factorial moment, $v_{sf(2)}$	0.0120	0.67	0.80
Multiplication, M	0.0020	0.01	< 0.01



Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE



$$S_{sf} = 678 (12) \text{ s}^{-1} \text{ g}^{-1}$$

$$S_{\alpha n} / S_{sf} = 0.039 (18)$$

Good agreement with:

Radiochemical analysis : $S_{sf} = 699 (28) \text{ s}^{-1} \text{ g}^{-1}$

Note: uncertainty radiochemical analysis > 4% (> factor 2)



Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE

Code	Library	S_{calc}/S_{exp}	S_{calc}/S_{exp}
		LIB	REC
ALEPH2	JEFF-3.3	0.94	0.95
SCALE	ENDF/B-VII.0	0.96	0.96
Serpent2 (2.1.29)	ENDF/B-VII.0	1.01	1.00
	ENDF/B-VII.1	1.02	1.01
	ENDF/B-VIII.0	1.02	1.01
	JEFF-3.1.2	0.93	0.92
	JEFF-3.3	0.97	0.96
	JEFF-3.3 (1)	1.09	1.08
	JEFF-3.3 (2)	1.01	1.00
	JEFF-3.3 (3)	1.01	0.99
	JEFF-4T1	0.98	0.97
	JENDL-4.0u (4)		1.06
JENDL-5.0		1.09	1.08

$$S_{sf} = 678 (12) s^{-1} g^{-1}$$

Good agreement with:

Radiochemical analysis : $S_{sf} = 699 (28) s^{-1} g^{-1}$

Theoretical calculations

Difference between LIB and REC data (~ 1%)

- (1) $\sigma(n,\gamma) = 0$ for ^{147}Nd
- (2) $\sigma(n,\gamma)$ for ^{147}Nd from JENDL-4.0u
- (3) $\sigma(n,\gamma)$ for ^{147}Nd from JEFF-4T1
- (4) No data available to calculate S_{sf}



Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE

Code	Library	S_{calc}/S_{exp}	S_{calc}/S_{exp}
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Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SGMENT SAMPLE

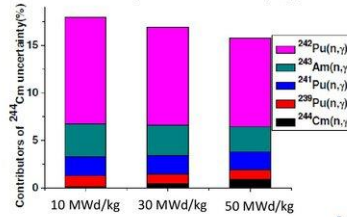
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JENDL-5.0	1.09	1.08	

$S_{sf} = 678 (12) s^{-1} g^{-1}$

Uncertainty theoretical estimation

Burnup	Relative uncertainty
10 MWd/kg	12.1 %
30 MWd/kg	11.1 %
50 MWd/kg	10.0 %

Contribution mainly from : $^{241}Pu(n,\gamma)$ & $^{243}Am(n,\gamma)$



Tiejun Zu et al., Annals of Nuclear Energy 94 (2016) 399

Training material



NEUTRON EMISSION: ABSOLUTE MEASUREMENTS OF SNF SEGMENT SAMPLE

Code	Library	S_{calc}/S_{exp}	S_{calc}/S_{exp}
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JENDL-5.0	1.09	1.08	

$S_{sf} = 678 (12) s^{-1} g^{-1}$

Some conclusions:

- Recommended decay data and neutron emission data not always adopted in evaluated data libraries
- $^{147}Nd(n,\gamma)$ cross section in JEFF-3.3 and ENDF/B-VIII.0 are too high (important for normalisation of PIE data)
- Fission yields for ^{148}Nd in JENDL-5.0 are too low
- $^{241}Pu(n,\gamma)$ and $^{243}Am(n,\gamma)$ cross sections require a re-evaluation (use of available experimental data)

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SUMMARY AND CONCLUSIONS

- The main observables of interest for a safe handling, transport, storage and disposal of SNF were identified and discussed
- A characterisation of SNF for these observables requires the inventory of some key nuclides with different characteristics
- The inventory of the key nuclides can only be obtained by theoretical calculations using depletion codes
- Uncertainty evaluation and propagation requires a good understanding of the measurement/calculation process
- The quality of the theoretical calculations strongly depends on the quality of the nuclear data and design and operational history of the SNF
- A performance assessment and uncertainty evaluation of the calorimeter installed at CLAB is required to improve its predictive power for the validation of depletion calculations
- Some key nuclear data (including their uncertainties) such as cumulative fission yields and neutron induced capture cross sections need to be improved to allow an accurate estimation of the main observables of interest including reliable confidence limits

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LEARNING OUTCOMES

After the completion of this training lesson, the participants should be able to

- understand the basic principles of uncertainty evaluation and propagation
- identify the observables that are important for the transport, handling, storage and disposal of spent nuclear fuel
- identify key nuclides determining these observables
- identify the different components involved in the theoretical calculations of the observables
- understand the importance of nuclear data and design characteristics and operational history of the fuel for an accurate theoretical estimation of the observables
- realise the need of accurate experimental data to validate theoretical calculations

Training material





KEY WORDS

- Bias
- Correlation matrix
- Covariance matrix
- Counting statistics
- Expanded uncertainty
- Monte Carlo simulations
- Normal (Gaussian) distribution
- Poisson distribution
- Random effect
- Sandwich formula
- Standard deviation
- Standard uncertainty
- Systematic effect
- Taylor development
- Uncertainty components
- Uncertainty propagation
- Actinides
- Alpha-decay
- Bateman equation
- Beta-decay
- Burnup
- Cooling time
- Cross section
- Decay heat
- Decay power
- Depletion codes
- Disposal
- Fission product
- Fission yields
- Gamma-ray emission
- Initial enrichment
- Irradiation history
- Light water reactor
- Neutron emission
- Neutron reactions
- Neutron induced capture reaction
- Neutron induced fission reaction
- Neutron transport
- Nuclear data
- Pressurised water reactor
- Recoverable energy
- Spent nuclear fuel
- Spontaneous fission

Training material




ADDITIONAL BIBLIOGRAPHY AND REFERENCES

Spent nuclear fuel observables and key nuclides

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- Schillebeeckx et al., "Characterisation of spent nuclear fuel by theoretical calculations and non-destructive analysis", see JRC114178 (2019)

Depletion codes

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ADDITIONAL BIBLIOGRAPHY AND REFERENCES

Radiochemical analysis and validation of depletion codes

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Non-destructive assay

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- B.D. Murphy and I.C. Gauld, "Spent Fuel Decay Heat Measurements Performed at the Swedish Central Interim Storage Facility", Report ORNL/TM-2008/016, Oak Ridge National Laboratory, February 2010
- Schillebeeckx et al., "A non-destructive method to determine the neutron production rate of a sample of spent nuclear fuel under standard controlled area conditions", JRC Technical Report EUR 30379 EN, 2020

Nuclear data

- Nuclear data libraries at JANIS NEA, https://www.oecd-nea.org/icms/pl_39910/janis
- Decay data, <http://www.nucleide.org/DDEP.htm>
- Nichols et al., "Handbook of nuclear data for safeguards: database extensions, august 2008", INDC-2453, INDC(NDS) – 0534

Training material



Appendix H. Slides of lecture B.3

B.3. Uncertainties associated with waste management routes

Prepared by Chris de Bock, ONDRAF/NIRAS



UNCERTAINTIES ASSOCIATED WITH WASTE MANAGEMENT ROUTES

Chris De Bock • WP 09



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

Date

Training material

1



KEY WORDS.

- Waste management
- Characterization
- Disposal routes
- Legacy waste
- Treatment and conditioning
- Shared solutions

Date

Training material



2



CONTENTS

- Learning outcomes.
- Introduction.
- Challenging wastes: background and waste types
- Legacy waste
- Early or delayed conditioning
- Shared solutions
- Summary and conclusions
- Used acronyms
- Bibliography and/or References.

Date

Training material



LEARNING OUTCOMES.

After the completion of this training unit/lesson, the students/participants should be able to:

- Understand why the waste management route of certain waste types is difficult to define
- Be aware of the waste types that are generally considered to be challenging
- Be acquainted with:
 - The double vicious circle entrapping legacy waste
 - The dilemma of early or delayed conditioning
 - The issues related to shared solutions (particularly for SIMS)

Date

Training material





INTRODUCTION

The backbone of radioactive waste management is the definition, for each waste type, of a route of activities from the generation of the waste to its disposal.

However, for certain waste types, the definition of a management route may be called *challenging*.

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CHALLENGING WASTES

- Within EURAD, uncertainties associated with waste management routes are dealt with in WP 9 “ROUTES” (see reference 1)
 - ROUTES focuses on *challenging wastes*
- Challenging wastes = wastes for which it is difficult to define an appropriate management route
- For challenging wastes, no complete management solution is available
 - One or more of the predisposal steps are missing (e.g. characterisation, treatment, conditioning)
 - Disposal strategy is not yet defined
- Reasons for difficulties to define an appropriate management route
 - Intrinsic properties of the waste (e.g. hazardous waste)
 - Uncertainties related to radiological or non-radiological characteristics (e.g. legacy waste)
 - Uncertainties related to waste behavior in certain waste management life-cycle steps (e.g. conditioning, disposal)
 - **No disposal strategy definition in turn impacts characterisation uncertainty (what to characterize + priorities)**
 - Small quantities of waste (typically for SIMS)
 - **Making certain technical options disproportionately expensive**
 - **A solution may be the sharing of infrastructure, mobile equipment or know-how**

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DIFFICULT MANAGEMENT ROUTE DEFINITION

- Key factors when choosing a waste management route
 - Waste-related factors: waste quantity, radiation level, half-life or radiotoxicity, chemical and physical characteristics (intrinsic and relevant to the interaction between waste and matrix), availability of a suitable treatment technology
 - Broad factors: availability of a repository, well-established WAC, regulatory considerations, public involvement
- Main blocking points for management route definition (from responses to ROUTES questionnaire)
 - Lack of disposal route (31%)
 - Characterization issues (22%)
 - Treatment or Conditioning issues (20%)
- ROUTES has identified 11 wastes types that are generally considered to be challenging wastes
- Per waste type, the areas of uncertainty per life-cycle phase have been investigated
 - Case studies play important role in ROUTES

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CHALLENGING WASTE TYPES

- Following challenging waste types were identified by ROUTES (see reference 2)
 1. Sludges
 2. Spent ion exchange resins
 3. Organic waste
 4. Bituminized waste
 5. Graphite waste
 6. Decommissioning waste
 7. Disused sealed radioactive sources
 8. Radium/Thorium/Uranium-bearing wastes
 9. Spent fuel
 10. Wastes containing reactive metals
 11. Wastes containing chemo-toxic substances

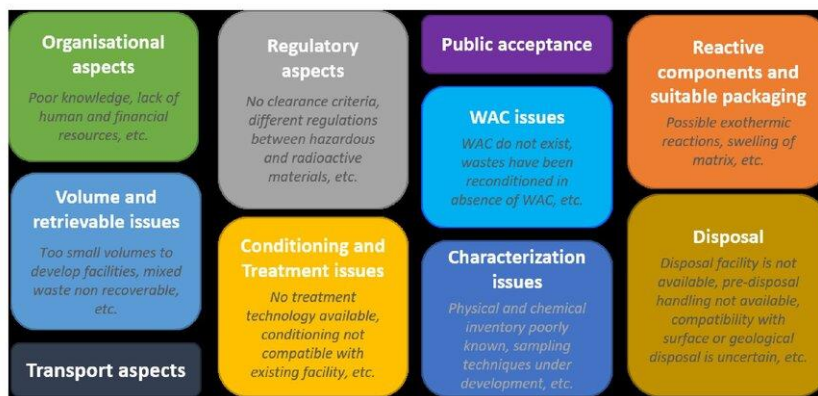
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UNCERTAINTY AREAS



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ISSUES COMMON TO LEGACY WASTES

- Root cause are characterization uncertainties
 - Missing information about origin
 - Missing information about radionuclide content
 - In some cases, waste streams have been mixed
- Related issues
 - Quantification of specific radionuclides or species
 - Indirect methods (e.g. C-14 in graphite waste)
 - Direct methods (e.g. activation products, complexing substances)
 - Strategy for the retrieval of the unconditioned waste

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THE DOUBLE VICIOUS CIRCLE ENTRAPPING LEGACY WASTES

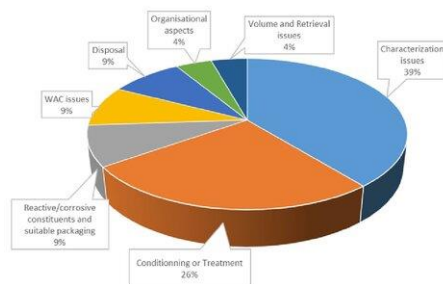


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THE EXAMPLE OF LEGACY SLUDGES



- Sludges are a common challenging waste
 - UK: sludge from degradation of Magnox SF in long storage
 - France: sludges from SF reprocessing and liquid effluent treatment
- Approaches to address challenges
 - Characterisation campaigns examining material sampled at different depths
 - In-depth examination of historical records (where available)
 - Requirements to recondition drums of corroded waste offer opportunities for further sampling and characterisation

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CHOOSING BETWEEN EARLY OR DELAYED CONDITIONING

- Root cause of this dilemma is the absence of an established disposal route
- Early conditioning
 - Encourages standardization, thus contributing to cost minimization
 - Requires close dialogue between all stakeholders (producers, WMO, regulator ...)
 - Requires stability of WAC
- Delayed conditioning
 - Leave options open, thus reducing the initial investment and the financial risk
 - Requires future retrieval and repackaging of potentially degraded waste

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SCHEMATIC REPRESENTATION OF THE DILEMA



Date

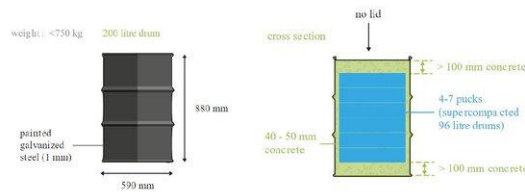
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EARLY OR DELAYED CONDITIONING - NETHERLANDS EXAMPLE CASE (1/3)

- National policy: interim storage ≥ 100 years followed by geological disposal
- Choice for early conditioning
- LILW in the Netherlands is mostly solid compactable waste
 - Solution based on compaction and cementation in 200-litre galvanized steel drums
 - Concrete for conditioning is made of blast furnace slag, cement, water aggregates and plasticizers
 - Waste characteristics are tentatively assumed based on expert judgement (organics, metals, plastics and others)



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EARLY OR DELAYED CONDITIONING - NETHERLANDS EXAMPLE CASE (2/3)

- Safety assessment and WAC for storage are stringent and considered to be bounding for safe disposal
 - WAC for transport and waste processing and storage have been established several decades ago
 - Ongoing disposability research to support continued improvements in WAC
 - Waste packages should be ready for disposal after storage period (no additional packaging)
 - National and shared disposal solutions are studied in the research programme. No site location has been identified.
 - Research is further advanced for disposal in clay than disposal in salt formation
 - Approach applied across a wide range of waste classes and categories (except VSLW and exempted waste)

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EARLY OR DELAYED CONDITIONING - NETHERLANDS EXAMPLE CASE (3/3)

- Waste packages are stored under dry conditions in a stack
 - Storage conditions considered to be mechanically severer than disposal conditions
 - Packages must also meet requirements for chemical and physical stability in case of flooding of the storage facility
- Monitoring and inspection to confirm stability of packages during storage
 - Every 15-20 years waste packages are checked,
 - Possible defects are repaired
 - Conditioning is adjusted if needed



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EARLY OR DELAYED CONDITIONING – UNITED KINGDOM EXAMPLE CASE (1/2)

- Sellafield Ltd has developed self-shielded boxes (SSBs) for storage of highly degraded Magnox fuels which have been or will be recovered from the legacy ponds and are not suitable for reprocessing.
 - Packaging of spent fuel in SSBs without matrix conditioning
 - Ductile cast iron walls provide radiological shielding and containment
 - Eight Filters allow hydrogen gas (from radiolysis and corrosion of metallic uranium etc.) to egress and oxygen to ingress
- Sellafield Ltd. has built new Interim Storage Facility, which could potentially hold the SSB-packages
- Packages might be disposed but waste could also be reconditioned relatively easily



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EARLY OR DELAYED CONDITIONING – UNITED KINGDOM EXAMPLE CASE (2/2)

- Benefits of this strategy:
 - Accelerated hazard reduction (emptying and decommissioning of ageing facilities)
 - Lower upfront capital investment
 - Flexibility in managing such wastes



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THE CONCEPT OF SHARED SOLUTIONS

- Shared solutions do not have to be purely technical. Shared solutions encompass
 - Research carried out and knowledge used
 - Technology developed and transferred
 - Facilities constructed and operated throughout all the waste life-cycle phases
 - Legal and institutional arrangements established
 - Processes of interaction between the stakeholders, including safety culture and governance issues
- Sharing solutions between countries has benefits
 - Make needed infrastructure available and exchange know-how
 - Share costs
 - However, there are also issues

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ISSUES RELATED TO SHARED SOLUTIONS

- Prerequisite to developing shared solutions is a common safety culture and a level playing field between participating countries.
 - If not: tendency to shift development and location of shared infrastructure to country with lowest standards
- Transparency must be established
 - Public access to information
 - Evidence-based decision-making
 - Effective public participation and access to justice
- Specific deliberative process should be developed
 - Including proper representation from local, national and multinational actors (not only officials)

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SHARED SOLUTIONS IN PRACTICE

- Shared solutions for disposal: still in its feasibility phase
 - EC-projects SAPIERR and SAPIERR2 have led to establishment in 2009 of the ERDO working group, transformed in 2021 into the ERDO Association
 - At present, only one agreement for shared disposal: Belgium and Luxembourg
- Shared solutions for characterisation, treatment and/or conditioning:
 - Many examples of implementation
 - Recycling/processing of metals, incineration of Danish waste in Sweden (EDF Cyclife)
 - Reprocessing of SF in France (ORANO)
 - Recycling/processing of metals in Germany (Siemelkamp)
 - Mobile facilities are highly valued (no location issue)
 - Conditioning of ion exchange resins (EDF Cyclife France has two mobile facilities)



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SHARED SOLUTIONS AND SIMS

- Constraints faced by SIMS:
 - Limited pools of expertise or know-how
 - Limited availability of infrastructure
 - Limited budgets for development of specific new solutions or for radioactive waste management in general
 - Often immature or absent disposal strategy
- Shared solutions would provide best added value for SIMS
 - Needed infrastructure becomes available and know-how is exchanged
 - Costs are shared

Greek case

- ~60 drums of cemented residues from research activities between 1970-1990
- Uncertainty over inventory and approach to characterise
- Evaluating options for deploying mobile characterisation facilities, drawing on experience and capabilities in other MS



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SUMMARY AND CONCLUSIONS

- Challenging wastes = wastes for which it is difficult to define an appropriate management route
- ROUTES has identified 11 wastes types that are generally considered to be challenging wastes
- Main blocking points for defining a management route
 - Lack of disposal route
 - Characterization issues
 - Treatment or Conditioning issues
- Characterization uncertainties are the root cause of the issues around legacy waste
 - Legacy waste risks being entrapped in a double vicious circle around characterization
- Absence of an established disposal route is the root cause of the early or delayed conditioning dilemma
- Sharing solutions between countries may have substantial benefits, particularly for SIMS
 - Make needed infrastructure available, exchange know-how, share costs
 - However, prerequisite is a common safety culture and a level playing field between participating countries

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USED ACRONYMS

- SF = spent fuel
- SIMS = small inventory member state
- WAC = waste acceptance criteria
- WMO = waste management organization

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BIBLIOGRAPHY AND/OR REFERENCES.

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2. V. Wasselin, M. Maitre, I. Kutina, *Overview of issues related to challenging wastes*, Final version as of 18.08.2022 of deliverable D9.5 of the HORIZON 2020 project EURAD. EC Grant agreement no 847593 (2022)

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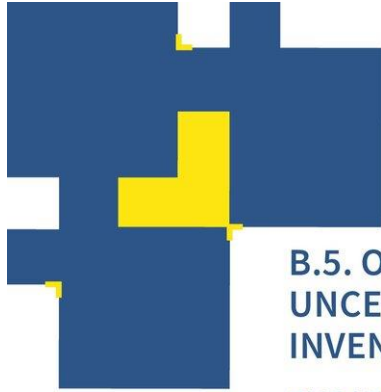
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Appendix I. Slides of lecture B.4

B.4. Options for the management of uncertainties related to the waste inventory

Prepared by Jeroen Mertens, Bel V



B.5. OPTIONS FOR THE MANAGEMENT OF UNCERTAINTIES RELATED TO THE WASTE INVENTORY

EURAD Training on Uncertainty Management

UMAN WP 4.2 – Part ‘waste inventory’ • Jeroen Mertens



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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KEY WORDS



Uncertainty management options waste inventory, radionuclide activity, chemical composition, physicochemical conditions in storage or disposal, sampling, measurements, conservative approach, proces understanding, monitoring

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CONTENTS

- Learning outcomes.
- Introduction.
- Uncertainties seen as most important to WMO/TSO/RE (survey)
- Uncertainties related to chemical composition (+ organics)
- Uncertainties related to physicochemical conditions in storage or disposal
- Uncertainties related to radionuclide activity (incl. vector)
- Conclusions.
- Bibliography and/or References.

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LEARNING OUTCOMES.

After the completion of this training unit, the participants should be able to:

- Understand and explain different ways of managing several of the waste inventory uncertainties; apply the different steps to be able to manage the uncertainty
- Understand the complexity of management of certain of the waste inventory uncertainties
- Be able to take home ideas of how to deal with some of the uncertainties related to waste inventory

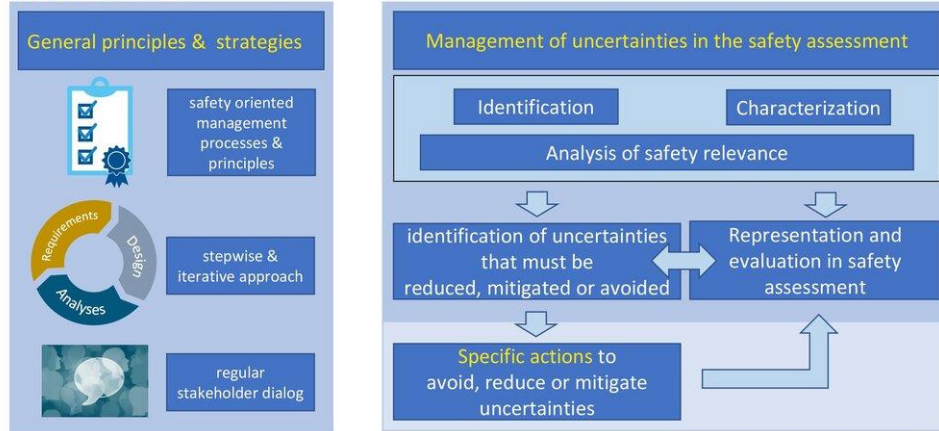
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INTRODUCTION: GENERIC STRATEGIES AND MANAGEMENT OPTIONS FOR UNCERTAINTIES



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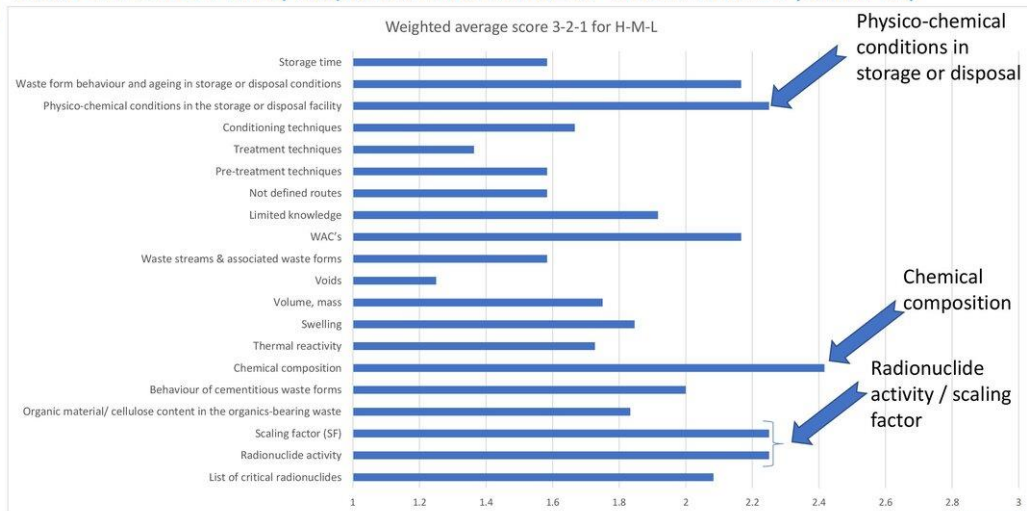
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SURVEY AMONGST WMO/TSO/RE ON IMPORTANCE OF UNCERTAINTIES (UMAN 4.2)





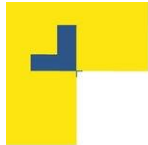
SUMMARY OF THE ANSWERS TO THE QUESTIONNAIRE UMAN 4.2 TASK

Most relevant uncertainties for RE, TSO and WMO according to answers to the survey:

- **Physico-chemical conditions in the storage or disposal facility**
 - Uncertainty in the evolution of the disposal system and its environment
- **Chemical composition (with a special attention to organic content)**
 - Uncertainty associated with initial characteristics
- **Radionuclide activity (including the scaling factor)**
 - Uncertainty associated with initial characteristics

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PHYSICO-CHEMICAL CONDITIONS IN THE STORAGE OR DISPOSAL FACILITY

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PHYSICO-CHEMICAL CONDITIONS IN THE STORAGE OR DISPOSAL FACILITY



Is the history of physico-chemical conditions in storage known?
 Are the physico-chemical conditions in the future disposal facility known?

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UNCERTAINTY



PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL - IDENTIFICATION

- Covers large domain:
 - **Many parameters** regarding physico-chemical conditions
 - As well **storage** as **disposal conditions**
 - Behaviour of the waste form in specific physico-chemical storage & disposal conditions dependent on the type of waste form (e.g. cementitious, bituminous), type of disposal,...
 - → **identification might be complex process**
- For identification, important to perform exhaustive analysis of the physicochemical conditions:
 - Storage, e.g. uncertainty in temperature (extremes), (relative) humidity, radiation, chemical environment
 - Disposal, e.g. pH, aggressive species, presence of water
- Identification generally done in the safety assessment through:
 - Literature survey/International REX
 - Development of system understanding
 - Features, Events & Processes or 'FEP'-screening

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL - CHARACTERIZATION

- Storage conditions:
 - Uncertainties could be characterized through identifying uncertainty on the **indoor climatic environment** (t°, humidity...), radiation field, chemical species that could be present in the storage facilities...
 - Conditions outside of the building (heat waves, cold, humidity) might also be useful to characterize the uncertainty, when they can be combined with knowledge on e.g. building isolation
 - Particular aspect could be past **accidents/incidents** with stored waste that could have an influence on the waste (e.g. fire, leaking roof, chemical issue?)



Picture © BP

What caused this? Humidity? Leaking water through roof? Substance leaking from other drum? Cl in the air? What is the situation inside?

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL - CHARACTERIZATION

- Disposal conditions:
 - Function of **disposal type and specific environment**. The conditions are not only determined by EBS or **host rock**, but also by **chemical elements** that are released from the waste that might change the physico-chemical conditions other wastes are in
 - Characterize the uncertainty through **literature survey**, **lab & demonstration experiments** (and applying statistical models), through **modelling** (scenario, sensitivity analysis) and **FEP analysis**, quantification by **expert judgement**



Disposal gallery @ Onkalo (Finland)

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL – ASSESSMENT OF THE SAFETY RELEVANCE

- Safety relevance assessed through different means:
 - Literature survey
 - Expert judgement
 - And mostly cited: **safety/performance assessment**, used e.g. to assess gas generation, pressure build-up, chemical evolution
- Relevance → can result in uncertainties regarding safety related properties and their evolution with time:
 - **Impact on physico-chemical properties** important for safety, such as mechanical stability or permeability of the waste form
 - **Impact on radionuclide release** from the waste form (increased leaching or degradation → more release)
 - **Impact on safety functions of the components surrounding the waste** (which could degrade engineered barriers): example is the swelling of the waste as a result of matrix degradation or corrosion of the package, mechanically damaging the EBS

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL– CLASSIFICATION AND ASSOCIATED ACTIONS (MANAGEMENT OPTIONS!)

- Unanimously identified as **to be reduced**
- Storage:
 - Assuring stable physico-chemically favourable conditions in the storage facility → uncertainty reduced to level it is not important any more. E.g. through climatic control in the storage facility (low moisture, constant temperature)
 - Reduce the uncertainty by **monitoring** physico-chemical conditions in the facility (t°, humidity...). In case not available, information from outside climatic conditions can be used in combination with knowledge on the building structure and isolation behaviour.

Avoided **To be reduced** Mitigated



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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL- CLASSIFICATION AND ASSOCIATED ACTIONS (MANAGEMENT OPTIONS!)

- Unanimously identified as **to be reduced**
- Storage:

Avoided
To be reduced
Mitigated

- In case no information is available, or in case waste has been moved and records on previous storage conditions are lost, uncertainty cannot as such be reduced. However, **physical conditions of the waste (package) itself** might give indication of past physico-chemical conditions and still reduce some of the uncertainty (e.g. presence or not of external corrosion of drums).



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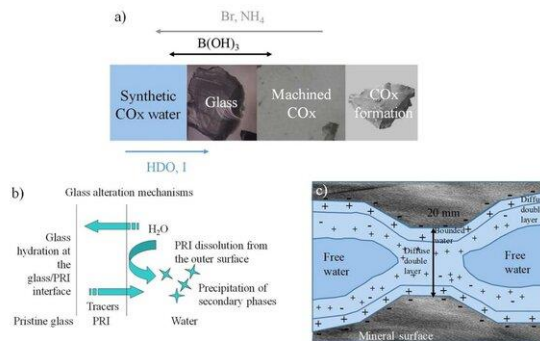


PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL- CLASSIFICATION AND ASSOCIATED ACTIONS

- Unanimously identified as **to be reduced**
- Disposal

Avoided
To be reduced
Mitigated

- Reduce through additional **experimental research** regarding physico-chemical conditions (lab and mock-up), e.g. in situ experiments for a DGR (mock-up installation, corrosion potential experiments,...). For surface disposal, construction of mock-ups of e.g. the cover to identify the chemical composition of water that could percolate through the facility after barrier degradation



Debure et al. 2019 – glass corrosion experiment

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL– CLASSIFICATION AND ASSOCIATED ACTIONS

- Unanimously identified as **to be reduced**
- Disposal

Avoided

To be reduced

Mitigated

Although not a way of reduction, several respondents marked that **robustness of the waste package and conditioning matrix against all kinds of physico-chemical challenges, reduces the importance of the uncertainty.** (e.g. corrosion resistant package, package that could compensate for a certain swelling of the waste form by using compressible materials, careful selection of materials used for conditioning,...)

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PHYSICO-CHEMICAL COND. STORAGE/DISPOSAL – REPRESENTATION IN THE SAFETY ASSESSMENT

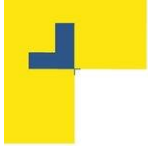
Different ways of representation exist:

- Via a **set of scenarios**, including **normal/reference and altered evolution** with e.g. supposed faster degradation of waste and/or EBS. The set of scenarios covers the range of uncertainties.
- Via carefully chosen **pessimistic (conservative) assessments of the reference scenario** (e.g. supposing oxidizing conditions in the surface disposal facility having the most unfavourable influence on retention properties in the waste form after degradation, or supposing metal drum is not a barrier)
- Via **probabilistic scenario assessments** where uncertainty is introduced in a probabilistic manner (e.g. probabilistic density functions)

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CHEMICAL COMPOSITION (WITH A SPECIAL ATTENTION TO ORGANIC CONTENT)

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CHEMICAL COMPOSITION - IDENTIFICATION

- Different issues regarding chemical composition:
 - Potential **impact of chemicals on the stability of the waste and waste packages, and EBS**
 - Chemical interaction with radionuclides **might alter transport properties** (e.g. complexing agents)
 - Hazardous materials might **impact waste handling and operational safety**
- Examples: chlorides, sulphates, nitrates, organic content (amongst which e.g. cellulose)
- Identified early in the development of the waste disposal program:
 - Particularly during **inventory development process or while developing WAC**
 - Identified through **systematic FEP-analysis** during safety assessments
 - Can also appear through **observation of absence of declaration** of detailed chemical composition: in particular observed for organic waste

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CHEMICAL COMPOSITION



picture © BP

Alkali-Silica reaction in cemented waste

Swelling of bituminous waste



picture © BP

Is the chemical composition known?

UNCERTAINTY

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CHEMICAL COMPOSITION (+ORGANICS) - CHARACTERIZATION

- Ideally, chemical composition of the waste should be well known, and thus related uncertainty quantitatively well characterized. However:
 - Regularly, **limited data is available** related to the mass of several species such as complexing agents
 - **Overestimated uncertainty** is happening if the WAC are **upper bounds** that need to be respected, and when only this information is available (example: cellulose “< 10kg”)
 - **Limited accuracy of measurements**, subsample analysis or the application of statistical methods on data induces uncertainty
 - **Waste generation history** may have varied or may vary, increasing uncertainty
 - (chemical) **evolution of the waste during storage** or in the facility (see also previous uncertainty)



Cellulose (©paragon protection)

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CHEMICAL COMPOSITION (+ORGANICS) - CHARACTERIZATION

- Usually, it is strived for to characterize uncertainty in chemical composition quantitatively, possibly with some additional safety margin
 - In case of limited data, **quantitative estimations based on expert judgement** including knowledge of the waste production could be used.

Homogeneous cementing of sludge from concentration of NPP effluents



© Westinghouse

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CHEMICAL COMPOSITION (+ORGANICS) – CHARACTERIZATION EXAMPLE

- Example WAC: limit of 15 g sulfates/kg waste form
- Waste drum -> no data on sulfate content
 - But: knowledge that the waste mass is 1206 kg
 - Based on the record, there is 7 kg of PVC in the waste as only potential source of sulfates
 - → so certainly <<< than 7kg sulfates/1206 kg waste or ~6 g/kg → upper bound uncertainty interval



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CHEMICAL COMPOSITION (+ORGANICS) – ASSESSMENT OF THE SAFETY RELEVANCE

- Safety relevance assessed through (quantitative) phenomenological descriptions of the repository performance (**performance assessment**)
 - E.g. assessment of the degradation of concrete barriers as a result of chloride and sulfate release from the waste form
 - Specific **sensitivity and uncertainty analysis** is used to determine the most relevant parameters and assess safety importance of the remaining uncertainties
- Good agreement across the community that uncertainty with respect to the following topics has the highest relevance for post-closure safety of the disposal facility:
 - Chemicals that lead to **degradation of barriers**, e.g. sulfate attack of cementitious materials, chloride attack of waste drums and concrete reinforcements, Alkali silica reaction
 - **Complexing agents that enhance radionuclide mobility resp. reduce retention**
 - Chemical components that **enhance release** (enhanced corrosion of metallic waste)
 - Chemical components that **influence speciation of nuclides** (e.g. form of C-14 compounds (organic/inorganic))
- Safety relevance of the uncertainty has been noticed to be **different between different repositories** (surface, near surface, deep geological)

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CHEMICAL COMPOSITION (+ORGANICS) – CLASSIFICATION AND ASSOCIATED ACTIONS

- Most uncertainty related to chemical composition → adequately **reduce** before licensing steps and in particular before operational license.

Avoided	To be reduced	Mitigated
---------	---------------	-----------
- Consensus that **appropriate WAC together with strategies for verification and enforcement** must be put in place to achieve adequate uncertainty reduction
- Furthermore, the following strategies can be used:
 - **analysis of specific waste streams** or specific waste samples to gather more detailed information about specific safety relevant materials
 - assure **best possible description of used materials** as soon as possible and *before* conditioning, and store the information accordingly
 - **optimize waste conditioning** and packaging where possible and appropriate. In some countries (e.g. Czech Republic), the WMO has specifically the possibility to control waste conditioning process by waste producer
 - assure that information on waste amounts, radionuclide inventory, and chemical composition is **updated regularly and appropriately**
 - **improve process understanding** e.g., by laboratory measurements on (site specific) sorption of organic and inorganic (C-14) compounds

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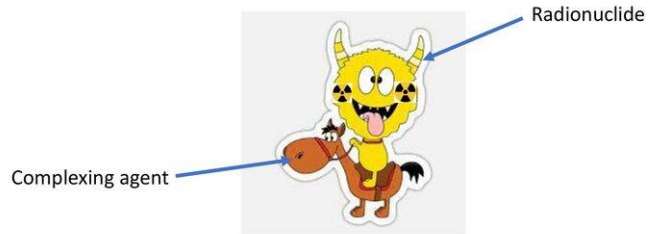
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CHEMICAL COMPOSITION (+ORGANICS) – REPRESENTATION IN THE SAFETY ASSESSMENT

- Mainly represented by conservative assumptions
- Sometimes specific assumptions are made, or altered evolution scenarios defined, to explore the impact of remaining uncertainty in the safety assessment (e.g. faster waste form degradation, no retention to account for complexing agents)



- Examples:
 - Example from Switzerland: For low and intermediate level waste in a cementitious near field → splitting into two waste groups postulated, where for one waste group we can assume better sorption behaviour; for the 2nd group, pessimistic sorption in a conservative way to account for complexing agents.

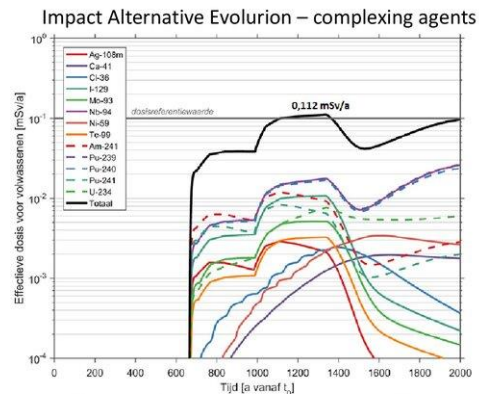
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CHEMICAL COMPOSITION (+ORGANICS) – REPRESENTATION IN THE SAFETY ASSESSMENT

- Examples:
 - Example from Lithuania: Remaining uncertainties in the program were addressed in safety assessment using a number of conservative assumptions associated with presence of organic C-14 compounds.
 - Example from Belgium: As there are many ways (and maybe even ways that might not be known) in which chemical species might degrade barriers or form complexing agents bypassing the retention barriers, special consideration is given to this in the safety assessment, based on FEP-screening. The assessment considers a specific Altered Evolution Scenario, in which it is supposed that 1% of the waste drums fails and contains complexing agents that had not been detected during verification of waste conformity. This results in a scenario where for 1% of the waste, all retention properties of the engineered barriers are reduced to 0. The results are then compared to the risk criterion to be applied to these scenarios.

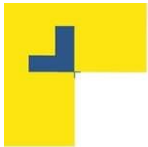


cAt safety case from Belgium, 2019, NIRAS/ONDRAF

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RADIONUCLIDE ACTIVITY (INCLUDING THE SCALING FACTOR)

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RADIONUCLIDE ACTIVITY (INCL. VECTOR) - IDENTIFICATION

- Wide variety of methods for radiological characterization: usually determined by measuring gamma emitters (easy to measure nuclides or ETM), and calculate difficult to measure nuclides (DTM) (mostly alfa/beta) by using scaling factors
- All of these methods subject to uncertainties, linked to:
 - **Representativeness** of samples
 - Measurement **accuracy**
 - **Model** uncertainty
- One more source of uncertainty is associated to the **limited knowledge on historical waste characteristics**
- Uncertainties identified while performing different activities in the development of the disposal program:
 - Conceptual phase: processing waste inventory
 - Performing radiological characterization
 - Updating of the inventory
 - Development of the program for demonstration of compliance with WAC

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RADIONUCLIDE ACTIVITY (INCL. VECTOR) - CHARACTERIZATION

- Two types of uncertainties:
 - Related to measurements
 - Characterized through **statistical methods on data**, multiple measurements to allow identify average, maximum/minimum
 - Aggregation of uncertainty on measurement, on conversion towards activity, and for the “difficult to measure” on the scaling factor
 - Related to models
 - Characterized through **probabilistic modelling**, taking account of sampling locations, or making assumptions (e.g. related to material impurities) to calculate average or maximum values.
- Important to keep in mind when characterizing: comprehensiveness of nuclide list, waste flow estimations, radionuclide composition in the waste, waste production process,...
- Uncertainty can be estimated at different levels: for each waste stream, waste family or for each waste drum/package and this for each radionuclide

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RADIONUCLIDE ACTIVITY (INCL. VECTOR) – ASSESSMENT OF THE SAFETY RELEVANCE

- Impact on conceptual planning, siting, construction, waste emplacement, operational and post closure safety, **WAC derivation, waste acceptance process** and instructions for safe waste management
- Radiological activity → linked to the waste management route → **uncertainty can impact the route**
- Underestimation of the activity → impact to workers dose, release and impact on human/environment in the long term or in case of accidents
- Relevance usually determined by safety assessment
 - If **bounding cases used regarding activity content in the safety assessment : significance is low**
 - If not, and in case of **nuclides that contribute to impact** (e.g. C-14) → medium to high significance
- Averaging effects of uncertainties on radionuclide activity when considering multiple packages (some overestimated, some underestimated) → global activity less uncertain
 - Importance of uncertainty when loading disposal facility → **can there be a systematic bias** giving rise to under/overestimation?
- **Compare upper bound uncertainty of radionuclide activity with the WAC** → if complies: no additional tests needed to confirm suitability and uncertainty has low relevance

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RADIONUCLIDE ACTIVITY (INCL. VECTOR) – CLASSIFICATION AND ASSOCIATED ACTIONS

- Classification: to be **reduced and/or mitigated** (mitigated mainly for historical waste)
- How to reduce:

Avoided	To be reduced	Mitigated
---------	---------------	-----------

 - Additional measurements on the waste
 - Scaling factors → regularly perform measurements and redefine/update the scaling factor
 - Activation calculations and verification by measurements
- Important to have close cooperation between WMO and waste producer
- In case uncertainty is difficult to reduce:
 - Use conservative approach in inventory definition
- Other aspects: improvement of waste management pre-sorting, uncertainty oriented research, development of novel waste characterisation methods,...

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RADIONUCLIDE ACTIVITY (INCL. VECTOR) – REPRESENTATION IN THE SAFETY ASSESSMENT

- Representation in different ways:
 - Use of reference values and upper bound values for the activity in different scenarios, in combination with conservative approaches where needed
 - In case radionuclide activity uncertainty is dealt with on the level of testing the compliance with the waste acceptance criteria, there is no need for special representation in the safety assessment

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MANAGEMENT OF UNCERTAINTIES RELATED TO RADIONUCLIDE ACTIVITY, AN EXAMPLE FROM THE BELGIAN SURFACE DISPOSAL FACILITY

- Surface disposal facility for Low and Intermediate Level Waste in Dessel – license expected < summer 2023.



Images © NIRAS/ONDRAF

- Uncertainty on the radionuclide activity → might result in exceeding waste acceptance criteria (WAC), specifically concentration limits per nuclide

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MANAGEMENT OF UNCERTAINTIES RELATED TO RADIONUCLIDE ACTIVITY, AN EXAMPLE FROM THE BELGIAN SURFACE DISPOSAL FACILITY

- Waste drum XX-245
 - Concentration limit for disposal: Nb-94: $7 \text{ E}+08 \text{ Bq/m}^3$
 - Maximum concentration Nb-94 as declared in a drum: $3,12 \text{ E}+07 \text{ Bq/m}^3 \rightarrow \text{OK?}$



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APPROACH

- Principle:
 - When [declared activity c° + uncertainty] does not exceed criterion → OK
 - When [declared activity c° + uncertainty] exceeds criterion → reduce uncertainty by additional tests to confirm radiological content (non destructive and/or destructive testing), because risk significant of exceeding limit. Certain % of the waste is tested.
- In practice:
 - For a given waste family → determine nuclides for which declared activity in waste is relatively close to the limit (e.g. < factor 100)
 - Determine uncertainty on activity concentration of these nuclides (see next slide)
 - Confront [declared activity c° + uncertainty] with limits and do according to the principle
 - Reiterate with nuclides excluded from uncertainty assessment that their exclusion is OK (e.g. exclusion > factor 100 is OK, given magnitude uncertainties obtained for other nuclides)

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DETERMINE UNCERTAINTY ON ACTIVITY CONCENTRATION

Uncertainty on the key nuclide

- In the example: Co-60 used as key nuclide for Nb-94 (both activation)
 - Nuclide is measured by using gamma spectroscopy
 - Uncertainties from measuring system, parameter settings, waste density, location of activity in the package,...
 - In our example: estimated to be an '**key nuclide uncertainty factor**' of 2 (which means that the activity can be 2x the declared activity c°)
 - Uncertainty factor due to key nuclide 'i' measurement uncertainty: $U_{f_{\text{KeyNucl}, i}}$

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DETERMINE UNCERTAINTY ON ACTIVITY CONCENTRATION

Uncertainty on the vector

- Determined based on data from previous validation campaigns, verification measurements
 - Larger than uncertainty on measured key nuclides
 - In our example: **vector uncertainty factor 20**
 - Vector uncertainty factor on nuclide 'j' = $U_{f_{Vector,j}}$
- Total uncertainty factor for nuclide 'j', determined using measurements on nuclide 'i':
 - $U_{f_{total,j}} = U_{f_{KeyNucl,i}} \times U_{f_{Vector,j}}$
 - In our case: $U_{f_{total,Nb-94}} = U_{f_{KeyNucl,Co-60}} \times U_{f_{Vector,Nb-94}}$
 - In our case: $U_{f_{total,Nb-94}} = 2 \times 20 = 40$

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DETERMINE UNCERTAINTY ON ACTIVITY CONCENTRATION

[Declared activity c° + uncertainty]

- $A_{j,+uncertainty} = A_{declared,j} \times U_{f_{total,j}} = A_{declared} \times U_{f_{KeyNucl,i}} \times U_{f_{Vector,j}}$
- In our case: $A_{declared,Nb-94} \times U_{f_{total,Nb-94}} = 3,12 \text{ E}+07 \text{ Bq/m}^3 \times 40 = 1,24 \text{ E}+09 \text{ Bq/m}^3 >$
limit (7 E+08 Bq/m³)
 - Ratio [declared activity c° + uncertainty]/Limit = $1,24 \text{ E}+09 \text{ Bq/m}^3 / 7 \text{ E}+08 \text{ Bq/m}^3 =$
~1,78 >>1

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DETERMINE UNCERTAINTY ON ACTIVITY CONCENTRATION

Radionuclide	Uncertainty factor on vector	Uncertainty factor on measurements	Total uncertainty factor
Nb-94	20	-	40
Co-60	1	2	40
Sr-90	4	-	16
Sn-126	4	-	16
Cs-137	1	4	4
Pu-238	4	-	24
Pu-239	4	-	24
Pu-240	4	-	24
Pu-241	4	-	24
Am-241	1	6	6

Collo	Am-241	Cs-137	Nb-94	Pu-238	Pu-239	Pu-240	Pu-241	Sn-126	Sr-90
XX-244	2,17E-01	8,80E-04	1,82E-01	7,90E-01	6,90E-01	2,61E-01	6,14E-02	7,80E-04	2,66E-02
XX-245	2,76E-01	6,49E-04	1,78E+00	2,5E-01	8,78E-01	3,33E-01	7,80E-02	5,76E-04	1,96E-02
XX-246	1,32E-01	5,98E-04	1,68E-01	2,21E-01	4,20E-01	1,60E-01	3,74E-02	5,28E-04	1,81E-02
XX-247	2,57E-02	5,01E-04	4,86E+00	2,36E-02	8,20E-02	3,10E-02	7,27E-03	4,44E-04	1,50E-02
XX-248	1,55E-01	8,08E-05	5,04E-03	1,42E-01	4,88E-01	1,85E-01	4,37E-02	7,16E-05	2,44E-03
XX-249	2,68E-01	3,49E-04	1,65E+00	2,46E-01	8,53E-01	3,22E-01	7,65E-02	3,09E-04	1,04E-02
XX-250	5,58E-01	3,93E-04	8,93E-02	5,14E-01	1,79E+00	6,72E-01	1,59E-01	3,49E-04	1,18E-02
XX-251	2,66E-01	1,07E-03	6,75E+00	2,44E-01	8,47E-01	3,21E-01	7,50E-02	9,48E-04	3,22E-02
XX-252	7,12E-02	2,22E-04	8,68E-01	6,54E-02	2,26E-01	8,58E-02	2,03E-02	1,97E-04	6,68E-03
XX-253	3,43E-01	8,90E-04	1,12E+00	3,14E-01	1,09E+00	4,14E-01	9,75E-02	7,88E-04	2,67E-02
XX-254	1,01E+00	2,61E-04	1,88E-01	9,36E-01	3,22E+00	1,22E+00	2,87E-01	2,32E-04	7,92E-03
XX-255	8,63E-03	7,66E-05	1,18E-02	7,98E-03	2,77E-02	1,05E-02	2,47E-03	6,80E-05	2,30E-03
XX-256	2,50E-02	4,25E-04	7,93E+00	2,30E-02	7,97E-02	3,01E-02	7,13E-03	3,78E-04	1,28E-02
XX-257	3,11E-02	4,03E-05	1,14E-01	2,87E-02	9,92E-02	3,76E-02	8,85E-03	3,58E-05	1,22E-03
XX-258	1,38E-02	6,42E-06	1,60E-02	1,28E-02	4,40E-02	1,67E-02	3,95E-03	5,68E-06	1,94E-04
XX-259	5,94E-03	5,76E-04	1,56E-01	5,47E-03	1,88E-02	7,14E-03	1,69E-03	5,08E-04	1,73E-02
XX-260	6,52E-02	9,61E-04	2,50E-01	6,06E-02	2,08E-01	7,86E-02	1,85E-02	8,56E-04	2,91E-02
XX-261	8,33E-01	5,25E-04	6,21E-02	7,68E-01	2,68E+00	1,01E+00	2,38E-01	4,64E-04	1,58E-02
XX-262	1,53E-01	2,41E-04	5,18E-01	1,41E-01	4,87E-01	1,84E-01	4,34E-02	2,14E-04	7,24E-03
XX-263	3,32E-03	1,69E-05	3,46E-01	3,04E-03	1,05E-02	4,00E-03	9,38E-04	1,49E-05	5,08E-04
XX-264	1,17E-01	1,27E-03	9,96E-01	1,08E-01	3,74E-01	1,42E-01	3,34E-02	1,12E-03	3,80E-02

(This is a real example, but not with the real data)

Declared activity c^e + uncertainty, divided by the limit exceeding limit is >1: red

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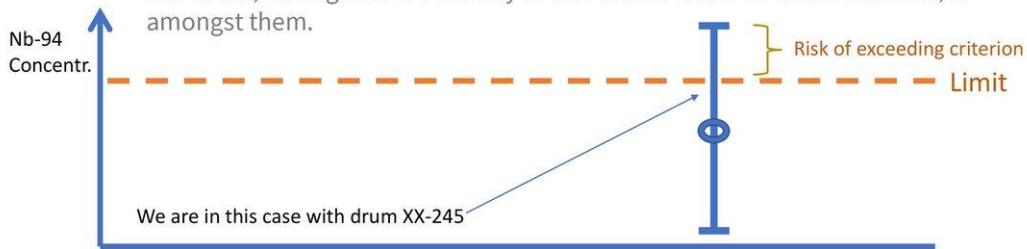
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DETERMINE UNCERTAINTY ON ACTIVITY CONCENTRATION

Several drums have risk of exceeding limit when taking into account uncertainty

- Our drum, having a Nb-94-activity of less than a factor 40 under the limit, is amongst them.



- Several of the drums will be selected for additional measurements (NDT/DT) in order to reduce uncertainty + confirm compliance

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CONCLUSIONS

- You have learned about different examples of uncertainties related to waste inventory, how to identify, characterize, assess the importance and manage them with the help of different management options. Examples on chemical composition, storage & disposal conditions, radionuclide activity were given.
- Almost each case of an uncertainty management is unique, because it depends on so many boundary conditions/parameters/...and so the way it is managed can easily differ from waste stream to waste stream, disposal concept to disposal concept, country to country. This course and the deliverables of UMAN related to this can hopefully be an inspiration for your cases, on how to deal with these uncertainties.

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JUST A VERY FINAL THOUGHT FOR TODAY...

Just sharing a view I always kept in mind when working with uncertainty. Something learned from a book.....



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JUST A VERY FINAL THOUGHT FOR TODAY...

Are all dinosaurs accounted for in the Jurassic park?
 → look for the 238 animals expected
 → yes they are all there – 238 found - OK

Total Animals		238		
Species	Expected	Found	Ver	
Tyrannosaurs	2	2	4.1	
Maiasaur	21	21	3.3	
Stegosaurus	4	4	3.9	
Triceratops	8	8	3.1	
Procompsognathids	49	49	3.9	
Othnielia	16	16	3.1	
Velociraptors	8	8	3.0	
Apatosaurs	17	17	3.1	
Hadrosaurus	11	11	3.1	
Dilophosaurs	7	7	4.3	
Pterosaurs	6	6	4.3	
Hypsilophodontids	33	33	2.9	
Euoplocephalids	16	16	4.0	
Styracosaurus	18	18	3.9	
Microceratops	22	22	4.1	
Total	238	238		

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JUST A VERY FINAL THOUGHT FOR TODAY...

Looking beyond what you expect to find

Total Animals		292		
Species	Expected	Found	Ver	
Tyrannosaurs	2	2	4.1	
Maiasaur	21	22	??	
Stegosaurus	4	4	3.9	
Triceratops	8	8	3.1	
Procompsognathids	49	65	??	
Othnielia	16	23	??	
Velociraptors	8	37	??	
Apatosaurs	17	17	3.1	
Hadrosaurus	11	11	3.1	
Dilophosaurs	7	7	4.3	
Pterosaurs	6	6	4.3	
Hypsilophodontids	33	34	??	
Euoplocephalids	16	16	4.0	
Styracosaurus	18	18	3.9	
Microceratops	22	22	4.1	
Total	238	292		

In uncertainty identification, characterization and management, it might be useful to keep looking beyond what is expected to find, and question from time to time the boundary conditions.... e.g. are all relevant nuclides in the vector? Could certain waste stream have been cross contaminated by other waste?

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BIBLIOGRAPHY AND/OR REFERENCES.

- EURAD UMAN 4.2 deliverable on waste inventory, in Deliverable D10.11: Study on management options for different types of uncertainties and programme phases
- NPJ materials degradation 3, Article number: 38 (2019) , “Mathieu Debure, Yannick Linard, Christelle Martin & Francis Claret. In situ nuclear-glass corrosion under geological repository conditions.
<https://www.nature.com/articles/s41529-019-0100-7>
- cAt-Surface disposal facility safety case – chapter 14 Long term safety assessment:
https://www.niras.be/sites/default/files/HS14_Veiligheidsevaluatie%20-%20langetermijnveiligheid.pdf
- Decree No. 377/2016 on requirements for safe radioactive waste management and on decommissioning of nuclear installations or workplaces of category III or IV, available at :
https://www.sujb.cz/fileadmin/sujb/docs/legislativa/vyhlasaky/377_Radioactive_Waste.pdf

And many more are in the EURAD deliverable D10.11

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Appendix J. Slides of lecture C.1

C.1 Views of Civil Society on Uncertainty Management

Prepared by Julien Dewoghélaère, NTW



VIEWS OF CIVIL SOCIETY ON UNCERTAINTY MANAGEMENT

EURAD Training on Uncertainty Management – Day 3

Julien Dewoghelaere (NTW) • WP 10 - UMAN



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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KEY WORDS



- Transparency and public participation
- Civil Society participation in research programmes
- Fruitful interactions between institutional/technical experts and civil society in uncertainty management linked to geological disposal
 - Pluralistic assessment and methodologies
 - Complex issues (geological disposal as a socio-technical tool)
 - Civil Society contribution to uncertainty management
 - Uncertainties related to human aspects

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CONTENTS



- Learning outcomes
- Introduction – Why having this lesson on civil society views related to uncertainty management ?
- Title 1 – Model of Civil Society (CS) involvement in EURAD and in UMAN project
- Title 2 – CS views on uncertainty management (global picture)
- Title 3 – CS views on uncertainties related to human aspects
- Title 4 – Identified methodologies to enable pluralistic management of uncertainties
- Summary and conclusions
- Bibliography and/or References
- Glossary

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LEARNING OUTCOMES



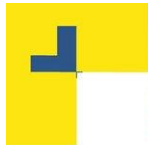
After the completion of this training lesson, the participants should be able to:

- Better understand the conditions and means for enabling meaningful interactions between experts and civil society in the field of nuclear research and uncertainty management
- Mobilise basic knowledge on CS views related to uncertainty management with a focus on uncertainty related to human aspects (based on the UMAN project results)
- Identify several methodologies that could facilitate a pluralistic management of uncertainties all along the disposal programme

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INTRODUCTION - WHY HAVING THIS LESSON ON CIVIL SOCIETY VIEWS RELATED TO UNCERTAINTY MANAGEMENT ?



- Involvement of stakeholders is essential at all stages of a Radioactive Waste Management (RWM) programme (and also it could be a legal requirement)
- Decisions have to be made in the presence of uncertainties. Dealing with uncertainties associated to disposal facilities is particularly challenging due to the long timescales (at least several generations)
- At the end of the process, some uncertainties will inevitably remain but it should be demonstrated that these uncertainties do not undermine safety arguments

In this context, it is necessary to implement a pluralistic management (diversity of actors, interdisciplinary and transdisciplinary) :

- To enable a mutual understanding of the uncertainties at stake (or at least a better understanding of the views of the involved actors)
- To enrich the research results of different views
- To address all the types of uncertainties that are embedded (technical and non-technical)

The implementation of such pluralistic management requires developing and experimenting innovative tools and methods

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TITLE 1

MODEL OF CIVIL SOCIETY INVOLVEMENT IN EURAD AND IN UMAN PROJECT

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INTERACTIONS WITH CIVIL SOCIETY IN EURAD: WHY ?

- Civil Society (CS) participants have specific concern on RWM safety, they are not research partners
- CS participants are involved in EURAD in the perspective of the UNECE Aarhus Convention which reinforces the requirements of Public access to information and participation in decision-making.
- One objective of EURAD is to allow interactions between all categories of actors : WMOs, TSOs, REs and Civil Society (“3+1 Dialogue”)
- Such interactions aim at improving mutual understanding of how and to what extent RD&D activities on RWM make sense and contribute improving decisions
- It also contributes to developing ideas, propositions and methodologies on
 - how to interact with Civil Society on scientific and technical results
 - how to deal with uncertainties

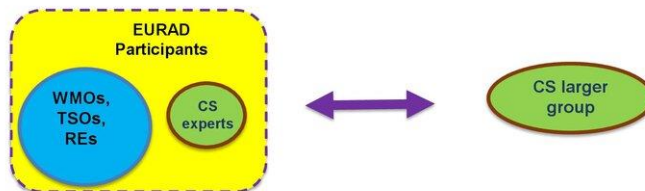
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ICS ACTIVITIES : HOW ? - DOUBLE WING MODEL

- **Model of pluralistic interactions developed and tested in previous projects (SITEX-II, JOPRAD)**
 - CS experts with technical and socio-technical background or/and experience on the involvement of CS in scientific and technical issues,
 - involved in EURAD activities through NTW (international association), translating scientific/technical results for exchanging with
 - A larger group of CS representatives (CSOs, representatives of local communities, individual experts)



CS Experts from Austria, Denmark, Finland, France, Hungary, Netherland, Slovakia, Slovenia, Sweden, United Kingdom

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CS LARGER GROUP – COMPOSITION

- D1.13 List of CS group members <https://www.ejp-eurad.eu/publications/eurad-d113-list-cs-group-members>
- Identification of potential members: 61 potential identified candidates coming from 25 countries
- Several categories of participants
 - 2 categories of actors and organisations:
 - European and national associations
 - local stakeholders (individuals and representatives of local communities, partnerships, local associations)
 - 22 members invited (according to the available resources to cover the physical participation) – group finalized in March 2020



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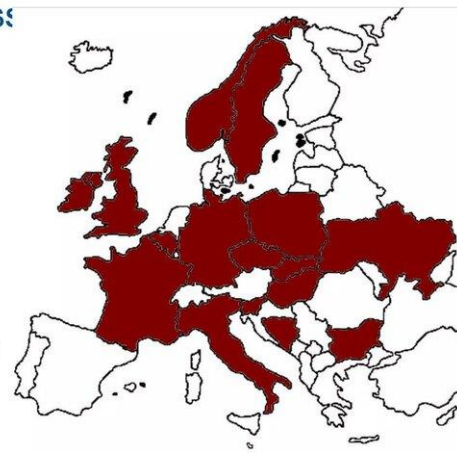
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CS LARGER GROUP – REPRESENTATIVENESS

- **A well-balanced group:**
 - Equilibrium between Western and Eastern countries
 - Quite well-balanced gender representativity (9 Women and 13 Men)
 - Good repartition between the categories of involved stakeholders (12 individual or/and local stakeholders and 10 national or/and European associations)
 - **15 countries are represented in the CS larger group:** Belgium, France, Germany, Italy, Norway, Sweden, United Kingdom, Bosnia and Herzegovina, Bulgaria, Czech Republic, Hungary, Poland, Slovakia, Slovenia, Ukraine

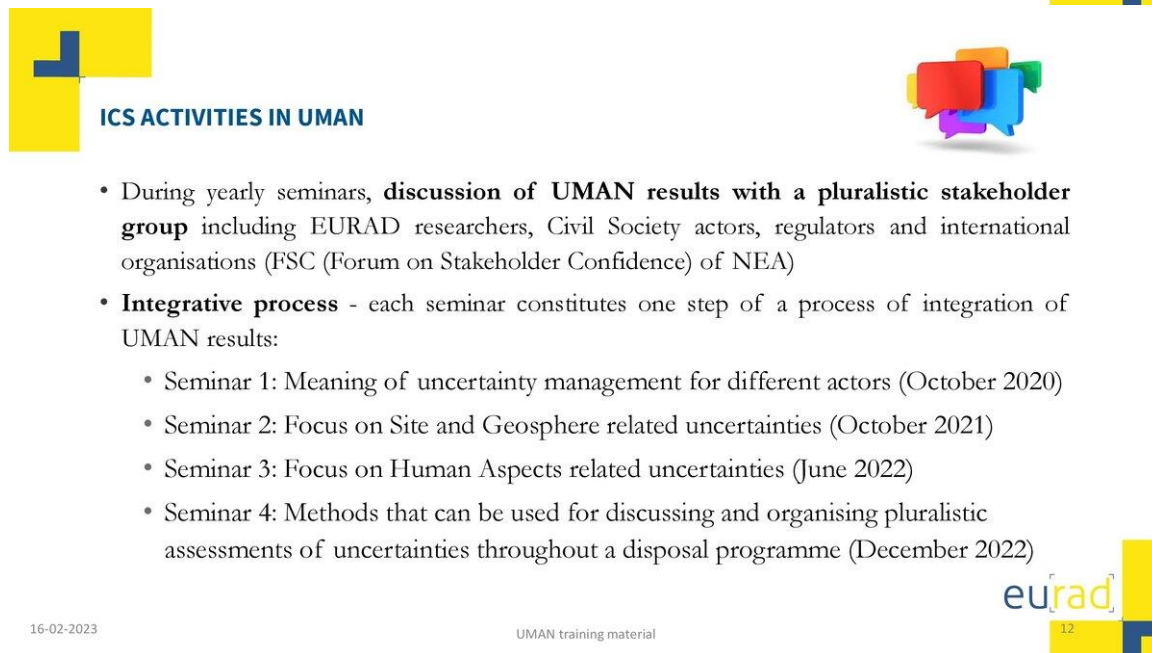
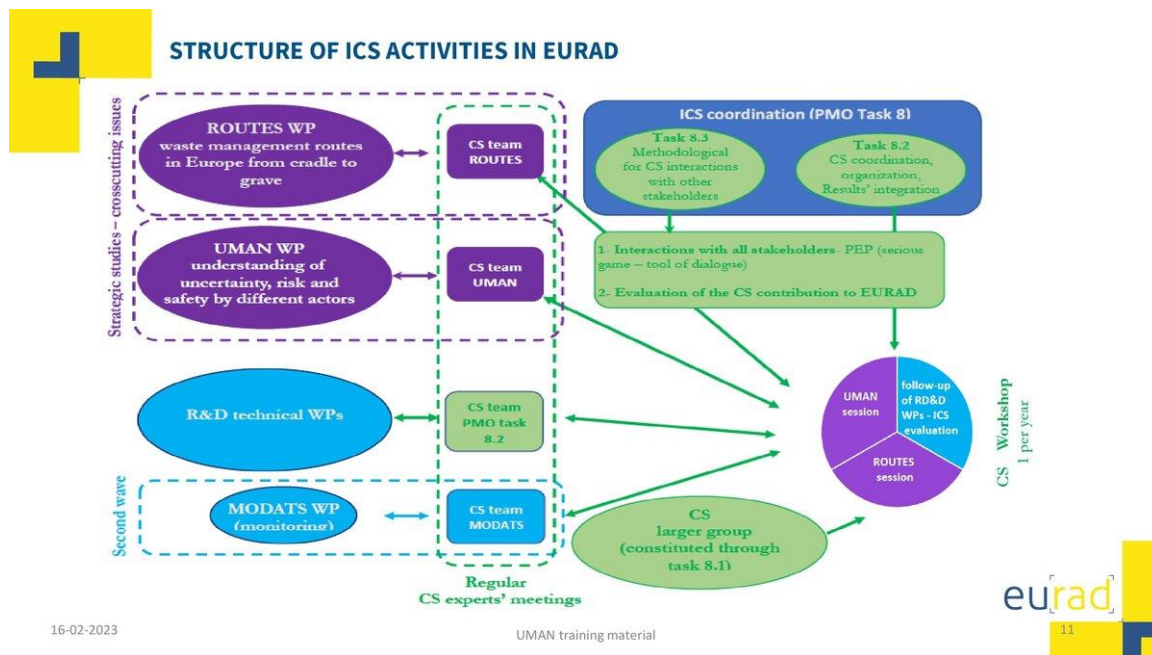


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TITLE 2

CS VIEWS ON UNCERTAINTY MANAGEMENT (GLOBAL PICTURE)

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RESULTS FROM UMAN SEMINAR 1 : GLOBAL PICTURE ON UNCERTAINTY MANAGEMENT



- Shared concerns by involved actors emerged from the pluralistic discussion in UMAN:
 - “Unknown unknowns” : how to address them? How to live with them? **How to be prepared to the unexpected?**
 - **Independence of expertise** : what does-it mean ? How it can be done in practice ?
 - Importance to consider uncertainty related to the process (governance issue), lot of **ignored knowns factors** (i.e. available knowledge one may not be aware of or fail to consider in one’s activities) to explore
 - At the general level, **agreement on the importance of uncertainty management in Safety Case**, differences will appear in concrete implementation (according to cultural contexts, role of the actors in the process, risk appetite)

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CS VIEWS ON UNCERTAINTIES RELATED TO GOVERNANCE



- **Continued independence of the authority is a precondition for effective Safety Case review.**
- Considerable uncertainties are attached to the continuity, availability and integrity of the governance framework (governing institutions and associated expertise) over long period of time.
- According to CS, this is already problematic in some EU member states.
- **Preserving independence of governing bodies cannot only be achieved on a national basis.**
- Safety standards depend on how democracy is handled. **The role of CS is very important in the upholding of the safety standards.** To increase confidence in the system, CS representatives should be involved in the decision-making of the governing institutions.
- There is also **uncertainty regarding the “independence” of the public** : compensation policies at local level can impede local inhabitants' awareness of safety priorities.

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SOCIETAL EXPECTATIONS ON SAFETY STANDARDS VARY OVER TIME



- **Uncertainties exist all along the successive phases of the repository program (and all along the whole RWM programme).** It is the reason why the decision to authorize (or not) implementation of GD cannot be a blank cheque.
- **The GD Safety Case has to evolve over time** due to science and interaction with society.
- A given option that is considered as safe today does not entail that future generations would have the same opinion. Would we accept today battlefield surgery conditions endured by soldiers on the 17th Century?
- **Evolution of the interpretation of the safety requirements should be anticipated** as a proof of the fairness of the decision-making process
- **Flexibility on the interpretation of safety standards is definitely a factor of confidence** on the long term.

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CS VIEWS - INVESTIGATING NON-TECHNICAL UNCERTAINTIES



- Placing RW into Deep Geology of a given place in the planet is, by no mean, solely a technical task.
- There is often some reluctance in the public with idea of reposing radioactive wastes in the geology, this is beyond the question of safety.
 - To incorporate GD in the geology of the place where I live is connected with the concrete relation I have with my territory and with other humans and non-human components of my environment.
 - This is linked with my vision of the planet, of the future, of the human destiny and role.
- Political and societal decision to authorize a GD is not a unidimensional consequence of the acceptance of the safety case by authorities. The safety case acceptance is only a pre-condition of the political decision.
- Achieving a decision to host or not a repository entails complex human and contextual aspects. Refusing to host a repository cannot be reduced to a NIMBY attitude. This should further be investigated by research.

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CS VIEWS - UNCERTAINTIES ON ALTERNATIVES



- **Will alternative scenarios and alternatives to the deep geological disposal be fully elaborated, reviewed and assessed beforehand as well as all along its implementation?**
- Will feedback on the operation and review of earlier decisions be performed, even at the later phases?
- Will there be a Plan B available if needed? (needs to be developed in parallel to Plan A)
- **Will the risk of abandoning the project before completion be taken into proper account?** (due to new knowledge on geology or technology, political crisis, end of budget ...)
- It is important for CS group members **that a failure to find a “good enough” plan will lead to a second, alternative Plan B. Then the uncertainty in the process itself that is perceived by the public may be reduced.**

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TOWARDS THE END OF TIME: SOME REFLECTIONS ON UNCERTAINTIES IN THE POST-CLOSURE PHASE



- **Will it be possible to recover the nuclear waste in the post-closure phase? For how long?**
- Will the possibility of full retrievability/recoverability be one of the determining criteria for choosing the proper type of disposal method?
- **What is needed is a characterisation of the needs for retrievability options and the corresponding criteria** in order to make it possible to plan for keeping these options open as long as possible.
- **Uncertainties on trans-generational aspects include information transfer from generation to generation**, the risk of memory and data loss, warning over time, the time perspective of surveillance (when can it be stopped?) and responsibilities, also after the responsible bodies have disappeared.

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CS VIEWS ON UNCERTAINTIES IN TRANSPARENCY AND PUBLIC PARTICIPATION (T&PP) – EFFECTIVE INTERACTION WITH CIVIL SOCIETY



- **THE OBJECTIVE OF ICS IN REGARD TO UNCERTAINTIES IS EFFICIENCY**
- The conditions for effective interaction with CS within R&D in RWM are linked to the **conditions for effective transparency in the governance on decision-making in RWM**. Transparency in decision-making is essential for the enduring and constructive engagement of CS.
- This is important for improving the **safety of RWM projects, facilities and repositories**. Effective transparency leads to better decision-making and can thereby increase confidence of CS in the quality and fairness of RWM decision-making processes.
- Effective Interaction with CS in R&D for RWM can **build up competence** that will allow for better understanding of issues of importance during decision-making.

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TITLE 3

CS VIEWS ON UNCERTAINTIES RELATED TO HUMAN ASPECTS

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THE LIST OF THE 10 UNCERTAINTIES RELATED TO HUMAN ASPECTS SELECTED BY UMAN PROJECT FOR FURTHER INVESTIGATION

- A: Process for the identification of a workable set of repository requirements
- B: Continuity of the waste management policy along political changes
- C: Robustness of the presently considered safety requirements with regard to the long term
- **D: Public acceptance of the repository at potentially suitable or projected locations**
- **E: Schedule to be considered for implementing the different phases of the disposal programme**
- F: Robustness of the safety case vis-à-vis sociotechnical factors
- **G: Reliability of monitoring results and safety analysis**
- **H: Adequacy of safety-related activities (in siting, design, construction, operation and closure) for the implementation of safety provisions**
- I: Robustness of safety performance vis-à-vis possible cyber-attacks or programming
- J: Availability of well-educated human resources and relevant experts in radioactive waste management along the repository lifetime until closure

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RELEVANCE OF PUBLIC ACCEPTANCE IN REGARD TO CS INVOLVEMENT



- **CS ASSERTION:** *In an open, democratic society, public acceptance of any GD is a GOAL IN ITSELF.*
- **As part of an UNCERTAINTY MANAGEMENT STRATEGY,** public acceptance can **CONFIRM** the quality of **safety solutions**. Conversely, public non-acceptance can be a way to **CORRECT** the wrong implementation of safety measures.
- Thus, public acceptance and non-acceptance can **change over time**, especially if safety measures are perceived as better or proven wrong.
- **ALSO:** Even if it is seen only as a mean to succeed with final disposal of RW – which could be the case by some of the research actors in EURAD– one could argue that the achievement of public acceptance is **one of the main reasons for the EURAD 3+1 dialogue itself**.

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CS VIEWS ON SCHEDULE TO BE CONSIDERED FOR IMPLEMENTING THE DIFFERENT PHASES OF THE DISPOSAL PROGRAMME 1/2



- **To what extent should uncertainty on schedule be considered as an uncertainty** for safety ?
- **Postponing decisions can be a condition for improving safety** (precautionary principle), taking appropriate time to manage unexpected events or uncertainties
- Differences of views between several authorities involved in the decision might be at the origin of disclosure of problematic aspects of safety
- **However postponing decision requires appropriate plan B**, in order to prevent waste packages ageing and rock and structures behaviour ...

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CS VIEWS ON SCHEDULE TO BE CONSIDERED FOR IMPLEMENTING THE DIFFERENT PHASES OF THE DISPOSAL PROGRAMME 2/2



- **Stepwise approach should provide due time in order to give proper attention** to setting the safety conditions for reaching a new step.
- **Stepwise approach should be embedded in a Long-term/Rolling Stewardship (RS) perspective** required by a multi-stakeholder and intergenerational governance, this could involve e.g. :
 - Scheduling a license renewal procedure every 10 years, in order to update the GD safety case review according to monitoring results and to incorporate updated stakeholder's views
 - There might be big and small steps, leading to foreseen or unforeseen direction.
- Flexibility (reversibility, retrievability, proper funding for alternative options should be part of RS provisions)
- Proper provisions to maintain safety while postponing is requested

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CS VIEWS ON NEW KNOWLEDGE



- The possibility of new knowledge is inherent to a safety analysis of a long-term process,
- New knowledge does not undermine the credibility of the safety review, on the contrary it does contribute to reinforce it
- The question is therefore the extent to which new knowledge can be integrated in the GD implementation in order to reinforce safety
- A structure has to be implemented to produce new knowledge and consider its relevance for the GD (esp. in the far future). Possibly the periodic safety reviews and the renewal of the licenses could be used as points in time to introduce and discuss new knowledge in a democratic , participatory way.
- Such a structure has to be linked to Rolling Stewardship.

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CS VIEWS ON ADEQUACY OF SAFETY-RELATED ACTIVITIES FOR THE IMPLEMENTATION OF SAFETY PROVISIONS



The implementation of an enlarged safety culture appears to be a precondition for ensuring continuity of safety related activities

The term 'Safety Culture' was first introduced in INSAG's *Summary Report on the Post-Accident Review Meeting on the Chernobyl Accident*, published by the IAEA in 1986,

Safety culture is that assembly of characteristics and attitudes in organizations and individuals which establishes that, as an overriding priority, nuclear plant safety issues receive the attention warranted by their significance.

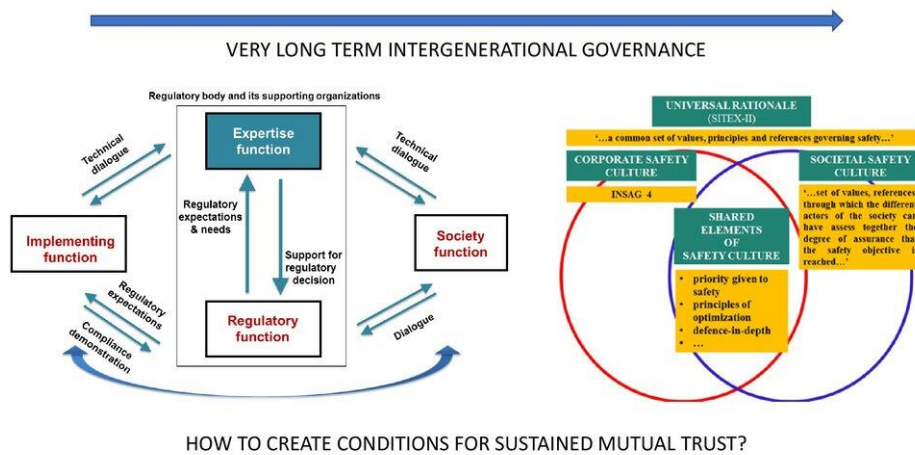
The Safety Culture is addressing the first circle of nuclear actors involved in nuclear safety: governments, regulators, operators, researchers & designers at institutional and individual levels


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AN ENLARGED SAFETY CULTURE TO SUPPORT VERY LONG TERM INTERACTIONS WITH SOCIETY (SITEX II) – FANC (TASK – LEADER)





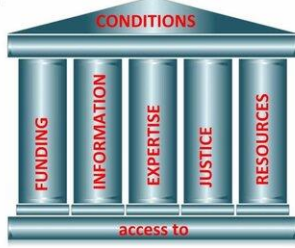
RESULTS: CONDITIONS AND MEANS

CS takes part in the decision-making process right from the **START**

There is **TIME** to consider and discuss the issue in depth before coming to a considered view

Public support can be generated through **trust**. Trust can be generated through **effective public engagement**.

DELIBERATIVE CHARACTER OF PUBLIC ENGAGEMENT



... trust should not be considered as a condition for the acceptance of a particular technical solution but as a condition for managing high complexity...[1]

ENGAGEMENT

↔

TRUST

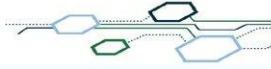

Engagement will build trust. But more trust is required to support engagement.

References:
[1] Luhmann, Niklas. "Trust and power, 1979." *John Wiley & Sons* (1979).

+

TRANSPARENCY

transparency of information, decision-making process, transparent reporting of participants' views...

TITLE 4

IDENTIFIED METHODOLOGIES TO ENABLE PLURALISTIC MANAGEMENT OF UNCERTAINTIES



IDENTIFICATION OF METHODOLOGIES AND PROCESSES TO ENABLE PLURALISTIC MANAGEMENT OF UNCERTAINTIES



During the UMAN seminars, thoughts and ideas have been raised regarding pluralistic management of uncertainties.

Task 5 gathered these ideas and conceptualised them in methodologies and processes enabling **multi-actors and multi-disciplinary** management of uncertainties in the frame of geological disposal.

The methodologies are divided in two categories:

- the generic methodologies identified during discussions
- methodologies that have been tested in the frame of EURAD research

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GENERIC METHODOLOGIES 1/3



Development of a **shared Safety culture** and its intergenerational transmission:

- Safety Culture is a very promising concept in order to sustain trustworthy interactions among the concerned categories of actors in the context of long-term RWM processes involving uncertainties but also need for flexibility according to progress & errors necessitating reorientation along the process
- Safety Culture is typically a sociotechnical concept
- Further research is needed to update the Safety culture concept to the specificities of RWM in order to:
 - Encompass the very long- term dimension of RWM processes
 - Include Civil Society at international, national and local level, along the development of the Safety Case within a Long-term/Rolling Stewardship perspective
 - Develop the specific requirement vis-à-vis Civil Society as a genuine contributor of the quality of RWM decisions, in the perspective of the Aarhus Convention

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GENERIC METHODOLOGIES 2/3 – LONG-TERM/ROLLING STEWARDSHIP AS A MANAGEMENT METHOD



Almost all uncertainties linked to RWM will have an unpredictable impact on future generations. Therefore, ensuring recoverability and keeping the memory of the repository alive could be important.

Implementation of a Long-Term/Rolling Stewardship culture could be a management strategy to investigate:

- **knowledge and responsibility for the nuclear waste is handed over in society from generation to generation in a structured way:** every 25 (?) years the nuclear waste is visited, checked, if needed repacked, the instructions are updated for the next generation
- it implies notably intergenerational transmission of information, empowerment of communities, cultural heritage, e.g., regular celebration around waste like Dutch case.

Rolling stewardship ideas need to be further investigated

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GENERIC METHODOLOGIES - 3/3



- Reinforcement of an **appropriate legal framework** enabling pluralistic interaction:
 - Aarhus Convention and its three pillars (access to information, effective public participation including report on ways public consultations are duly taken into consideration, access to justice if the two other pillars are denied)
 - Access to resources for enabling effective public participation as indicated in the BEPPER report
 - Prescriptive EU directives (e.g., Article 10 Transparency of RWM directive, promotion, and enhancement of public participation)
 - International recommendations and guidance (e.g., FSC, IGSC)
- Implementation of a **stepwise approach** including notably:
 - intergenerational safety case review
 - reversibility/Retrievability/ Recoverability principles
 - a continuous knowledge management

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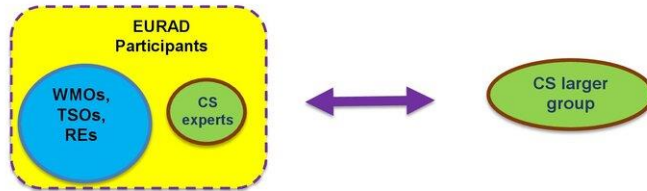




METHODOLOGIES TESTED IN THE FRAME OF EURAD 1/2



- **Double Wing Model** in the frame of research
 - for translating technical knowledge to enable a larger public to understand it and make up their own minds on the topic,
 - for problematizing socio-technical issues
 - Could the double wing model be extended to other situations? (e.g., follow-up of Geological Disposal implementation for instance) On what conditions?



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METHODOLOGIES TESTED IN THE FRAME OF EURAD 2/2



Pathway Evaluation Process (PEP) approach: concrete cases to organize pluralistic discussions on uncertainties on the same footing.

- tool of dialogue (designed as a serious game) developed under the frame of the SITEX.network that enable multi-actors' discussions in the field of radioactive waste management
- Methodologies could be used in in different formats and in different contexts:
 - E.g. in UMAN - Discussion in small groups are based on concrete cases illustrating the issues to be discussed (uncertainties in our case). The link with concrete issues enables all actors to enter the discussion on the same footing.

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GAME MECHANISM – PEP METHODOLOGY 1/2

- Elaboration of scenarios to test the robustness of a pathway :
one testing condition card (TC) associated with two Evaluation Criteria cards

▲	???	???
Political upheaval	Risk of abandonment	Confiscation from future generations
A major political change affects the decision-making process.	To what extent is the pathway vulnerable to possible abandonment in uncontrolled conditions, before reaching a Safe Terminus?	What are the unavoidable consequences of the pathway for future generations? What are the issues to be addressed by the successive generations? What margin for effective decision does the pathway leaves them?
<i>E.g.:</i> <ul style="list-style-type: none"> The split of the country jeopardises its RWM strategy. During internal troubles, existing storage site falls in the hands of separatists. A neighbouring country obtains that a facility is closed / moved. 		
Disruptive Events B5	Governance Quality Y9	Values & Ethics Z3

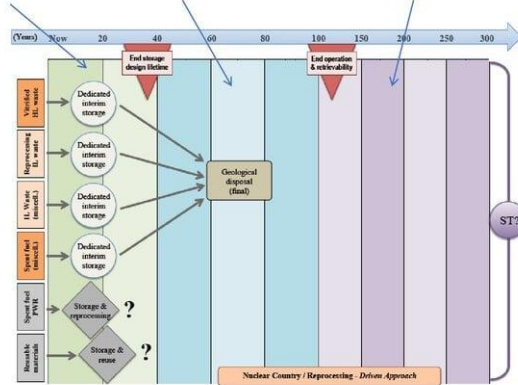
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GAME MECHANISM - PEP METHODOLOGY 2/2

- The TC card is positioned at a specific period:

Now/few decades Mid-Term/hundred year Long term/ few centuries





KEY MESSAGES REGARDING PEP TOOL



- PEP is not a tool to choose between approaches. The main aim is to allow a pluralistic discussion on the way to secure safety of humans and the natural environment through different strategies that have all advantages and disadvantages.
- PEP allows discussing a broad range of issues and envisioning situations and solutions participants may not have thought of.
- PEP helps the players to grasp the complexity of RWM that is considered here as a socio-technical issue, not only a technical one.
- PEP discussions emphasize the importance of transversal elements (to have in mind in all the pathway), notably institutional structure and background, meaningful public participation, pluralistic expertise, availability of financial resources, monitoring and memory in long-term horizons.

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SUMMARY AND CONCLUSIONS 1/2



- Interactions between experts and civil society aim at improving mutual understanding on uncertainty, contribute to the development of a shared safety culture and therefore contribute to improve R&D results and safety (at the end of the process).
- CS actors involved in EURAD have a strong interest to maintain independence of expertise, to reinforce research on uncertainties related to “unknown unknowns”, governance issues (including reversibility, transparency and post-closure) and non-technical uncertainties in general
- Public acceptance can be seen as an uncertainty management strategy: a complementary way to ensure the quality of the system (not necessarily as a human uncertainty)
- Postponing decisions can be a condition for improving safety (precautionary principle), taking appropriate time to manage unexpected events or uncertainties
- New knowledge does not undermine the credibility of the safety review, on the contrary it does contribute to reinforce it

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SUMMARY AND CONCLUSIONS 2/2



Methodologies and processes can be implemented to enable **multi-actors and multi-disciplinary** management of uncertainties in the frame of geological disposal :

- The implementation of an enlarged safety culture appears to be a precondition for ensuring continuity of safety related activities and for supporting an intergenerational, multi-actors management of uncertainties related to GD
- Implementation of a Long-Term/Rolling Stewardship culture could be a management strategy for ensuring intergenerational transmission of information, empowerment of communities, cultural heritage
- Reinforcement of an appropriate legal framework is necessary to enabling pluralistic management of uncertainty
- Double Wing Model in the frame of research is a way for translating technical knowledge in order to enable a larger public to understand technical results and make up their own minds on the research topic
- Development of dialogue tool (under the format of serious game) could facilitate the development of pluralistic management of uncertainties

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BIBLIOGRAPHY AND/OR REFERENCES



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- BEPPER Report: transparency in Radioactive Waste Management, December 2015: <https://www.nuclear-transparency-watch.eu/a-la-une/new-publication-bepper-report.html>
- EURAD Deliverable D 1.13: List of members of the Civil Society group, Mars 2020 : <https://www.ejp-eurad.eu/publications>
- Including civil society in R&D projects on Radioactive Waste Management: Interactions with Civil Society (ICS) in EURAD, BASE Symposium, November 2021: <https://sand.copernicus.org/articles/1/247/2021/sand-1-247-2021.pdf>
- EURAD Deliverable D1.14 : Mid-term evaluation of the ICS activities and experimental model of interaction between EURAD participants and Civil Society, April 2022
- EURAD Lunch and Learn Session on PEP methodology: <https://www.ejp-eurad.eu/news/recording-ll-pluralistic-tool-dialogue-rwm-pathway-evaluation-process-pep>

Future planned publications:

- Multi-actor dialogue on managing site and geosphere uncertainties in a radioactive waste management programme: lessons from the UMAN project (EURAD), WM2023 Conference, Phoenix Arizona (February-March 2023)
- EURAD UMAN Deliverable D 10.13: Understanding of uncertainty management by the various stakeholders, April 2023
- EURAD UMAN Deliverable D 10.14: Pluralistic analysis of site and geosphere uncertainty, April 2023
- EURAD UMAN Deliverable D 10.15: Pluralistic analysis of humans aspects related uncertainty, June 2023
- EURAD UMAN Deliverable D 10.16: Methods that can be used for discussing and organising pluralistic assessments of uncertainties throughout a disposal programme , June 2023

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GLOSSARY

- CS = Civil Society
- ICS = Interaction with Civil Society
- NTW = Nuclear Transparency Watch
- RE = Research Entities
- R&D = Research and Development
- RWM = radioactive waste management
- TSO = Technical Support Organisation
- UMAN = Uncertainty Management multi-Actor Network (project in EURAD programme dedicated to uncertainty management and interactions between the different types of actors for reaching a mutual understanding)
- UNECE = The United Nations Economic Commission for Europe
- WMO = Waste Management organisation



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Appendix K. Slides of lecture C.2

C.2 Characterization and significance of uncertainties related to human aspects

Prepared by Jeroen Bartol, COVRA



CHARACTERIZATION AND SIGNIFICANCE OF UNCERTAINTIES FOR DIFFERENT CATEGORIES OF ACTORS

Uncertainties related to human aspects.

Dr. Jeroen Bartol • WP 10 subtask 3.4



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593

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KEY WORDS.

- **Uncertainties related to humans**
- **Uncertainty list**
- **EURAD UMAN**
- **UMAN work package**

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CONTENTS

- Learning outcomes.
- Introduction.
- Before we start : phases.
- The process.
- Important uncertainties.
- Uncertainties about uncertainties.
- Summary and conclusions.
- Bibliography and/or References.

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LEARNING OUTCOMES.

After the completion of this training lesson, the students/participants should have an idea about the uncertainties identified in WP 10 subtask 3 and the 10 most important (as identified in the workgroup) uncertainties, their impact and how to deal with them. They will also learn how they can apply the method used in WP 10 subtask 3.4 to identified uncertainties and what they shouldn't do.

The uncertainties.

The ten most important ones, their impact and how to deal with them.

How to identify uncertainties.

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Over a period of 3 - 4 years, we Jean-Noël DUMONT (ANDRA), Nadja ZELEZNIK (EIMV), Jeroen BARTOL (COVRA), Erika HOLT (VTT), we have worked on this subtask. In this presentation, I will go deeper into this work package. More specifically, the process and the result.

The report has not been published yet but is very close to be finished (currently in review).



Deliverable 10.8: Views of the different actors on the identification, characterization and potential significance of uncertainties related to human aspects
Work Package WP10 UMAN, Subtask 3.4

The project leading to this application has received funding from the European Union's Horizon 2020 research and innovation programme under grant agreement No 847053.



Cover of the report of Dumont et al., (2023)

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The time period we are considering for the uncertainties. Note that we miss the need for action, disposal concept etc. They are before the site selection.

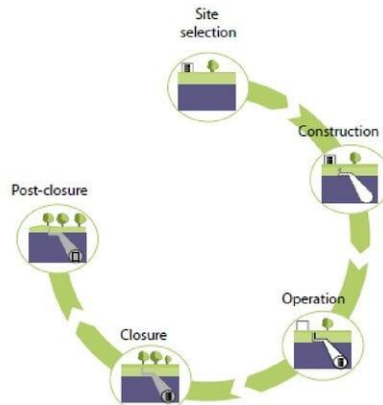


Figure from Verhoef et al., (2017)

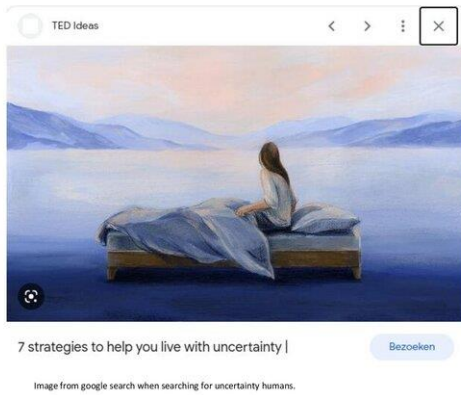
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INTRODUCTION | IMPLICATIONS AND HOW TO DEAL WITH IT



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BEFORE WE START | PHASES

Before we start, a word of caution. In our report, we have used **different names** for the phases. A reason for the discrepancies is that they have been established with categories of human activities in mind (**design, siting, construction...**), whereas the EURAD phases are defined as successive periods of time, even if their labelling refers to the main activity within each period.

Stage of the repository used for uncertainties identification	Stage Identification number	EURAD phase number	EURAD phase and theme
Need for action	0	Phase 11	Initiation: Policy, framework and programme establishment Theme 1: National Programme Management
Disposal concept development	1a	Phase 22	Site Selection: Site(s) identification and selection, Generic assessment of options Site Requirements and waste inventory Themes 2 (Pre-Disposal), 3 (EBS), 4 (Geoscience), 5 (Disposal facility design and optimisation), 7 (Safety case)
Site evaluation and detailed design	1b-2	Phase 22 Phase 33	Site Selection: Site(s) identification and selection, Site characterisation: Underground investigations and site confirmation, Site Requirements, Evaluation Theme 6 (Siting and licensing) Site Characterization & Selection
Construction	3	Phase 44 Phase 55	Construction: Facility construction, Operations and Closure: Facility operation and closure (facility construction and construction work carried out during operation and closure) Theme 3 (EBS), 5 (Disposal facility design and optimisation) and 7 (Safety case)
Operation	4a	Phase 55	Operations and Closure (except final closure) Theme 1 (National Programme Management), 5 (Disposal facility design and optimisation) and 7 (Safety case)
Closure	4b	Phase 55	Operations and closure (subphase related to final closure) Theme 1 (National Programme Management), 5 (Disposal facility design and optimisation) and 7 (Safety case)
Post-closure – Indirect oversight	5a	Phase 5	Post-closure phase (first subphase)
Post-closure – No oversight	5b	Phase 5	Post-closure phase (second subphase)

Dumont et al., (2023)

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The process | Step 1 Brainstorm



The process | Step 2 Reduction



The process | Step 3 Questionnaire

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THE PROCESS | STEP 1 IDENTIFICATION

A structured brainstorming within the expert group (representatives from 2 waste management organizations (WMOs), 1 research entity (RE) and 1 civil society, (CS), to identify as completely as possible the relevant uncertainties. The result was a long list of uncertainties (58) that are related to the humans. For the structured brainstorm sessions, we used the different phases of a repository life cycle. This was our inspiration.

When published, you can find the entire list of uncertainties online.

		Identification		Selection			
2.9	3	Management system safety culture	Governance	How well the safety of operation is assured from the management and other societal points of view?	Who will take part in operation? Will there be an independent body to manage management?	Medium	Similar to the safety culture within the company, at national level. Stresses the need for independent oversight.
4.1.1	4.1	Robustness of the safety case vis-à-vis socio-technical factors	Socio-technical	Are human and organisational factors together with technical factors properly taken into account in the safety case? How are political uncertainties affect licensing?	How is efficient communication ensured between the different experts? Who has expertise to have an overall picture of the safety? How do political uncertainties affect licensing? Is the possibility of combination/errors properly addressed?	High	Socio-technical i.e. interrelationships of societal, organisational and technical aspects create emerging risks and surprises. They play relevant role in safety case and licensing, and therefore, it is necessary to understand, identify, mitigate and tackle with these socio-technical factors.
4.1.2	4.1	Reliability of monitoring results and safety analysis	Socio-technical	Will new knowledge, insights or monitoring techniques reveal deficiencies that need to make corrective measures?		High	Consequences of weaknesses in monitoring techniques may have safety significance, even though it can be assumed that weaknesses in monitoring techniques will be noticed early enough.
4.1.3	4.1	Adequacy of activities in the implementation of operational (and long term) safety provisions	Human and organisational factors	How can changes in organisation and safety culture affect the safety of the operation phase?	How is it ensured that knowledge management is taken care of? How is it ensured that people have got adequate training?	High	Human and organisational aspects are inherently involved in the management of operation phase and in any reviews regarding operation, resources and technology. Furthermore, subcontracting of services and work complicates the management of human and organisational aspects. Socio-technical systems view emphasizes the need for seeing technical and organisational aspects simultaneously (Thorne and Stanton 2014).
4.1.4	4.1	Robustness of operational safety in a possible societal disruptions	Societal	How could societal disruptions affect safety?		Low	Societal disruption can have severe impacts on operation.
4.1.5	4.1	Robustness of safety in a possible cyber attacks or programming	Technical	How could cyber security and security aspects affect safety?	How should organisations take into account the possibility of intentional actions and errors?	High	Resilient controls, programming and related cybersecurity risks and uncertainties are worth of further investigations.

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THE PROCESS | STEP 1 IDENTIFICATION

Subsequently each of the uncertainty was assigned an specific category that was relevant in the identification of human related uncertainties. These categories are the following: **Societal, Political, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.**

Identification				Selection			
Id	Ref	Origin of uncertainty	Typology	General question	Examples of specific questions	Priority	Rationale for rating
5.9	3	Management systems; safety culture	Societal	How will the safety of operation be assured from the management and other societal aspects of events?	Who will take part in operations? Will there be an independent body to oversee management?	Medium	Similar to the safety culture within the company, at national level. Stresses the need for independent oversight.
4a.1	4a	Robustness of the safety case vis-à-vis socio-technical factors	Socio-technical	Are human and organizational factors together with technical factors properly taken into account in the safety case? How can political uncertainties affect licensing?	How is efficient communication ensured between the different experts? Who has expertise to have an overall picture of the safety? How do political uncertainties affect licensing? Is the possibility of unintentional errors properly addressed?	High	Socio-technical is, interconnectiveness of societal, organizational and technical aspects create emerging risks and surprises. They play relevant role in safety case and licensing, and therefore, it is necessary to understand, identify, mitigate and tackle with these socio-technical issues.
4a.2	4a	Reliability of monitoring results and safety analysis	Socio-technical	Will new knowledge, insights or monitoring techniques reveal deficiencies that need to make corrective measures?		High	Consequences of weaknesses in monitoring techniques may have safety significance, even though it can be assumed that weaknesses in monitoring techniques will be noticed early enough.
4a.3	4a	Adequacy of activities in operation for the implementation of (operational and long-term) safety provisions	Human and organizational factors	How can changes in organization and safety culture affect the safety of the operation phase?	How is it ensured that knowledge management is taken care of? How is it ensured that people have got adequate training?	High	Human and organizational aspects are inherently involved in the management of operation phase and in any decisions regarding operation, resources and technology. Furthermore, outsourcing of services and work complicates the management of human and organizational aspects. Socio-technical systems view emphasizes the need for using technical and organizational aspects simultaneously (Harvey and Stanton 2014).
4a.4	4a	Robustness of operational safety vis-à-vis possible societal disruptions	Societal	How could societal disruptions affect safety?		Low	Societal disruption can have severe impacts on operation.
4a.5	4a	Robustness of safety vis-à-vis possible cyber-attacks or	Socio-technical interface; unintentional (e.g. programming)	How could cyber security and security aspects affect safety?	How should organizations take into account the possibility of intentional actions and errors?	High	Remote control, programming and related cybersecurity risks and uncertainties are worth of further investigations.

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THE PROCESS | STEP 1 EXAMPLES

Societal, Political, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.

Societal relates to or involves society. Societal concerns /problems/values Examples of societal questions relevant for the nuclear waste management context would include social acceptance of nuclear waste management, **social media**, and **societal support for education of experts in the nuclear waste management**. Other examples for societal problems that could affect nuclear waste management are **wars**, **distrust** in institutions, and increase of right- and left-wing extremists.

Identification				Selection			
Phase	Object of uncertainty	Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
1b-2	Urgency of the availability of a repository	Possibility of societal disruptions enhancing the risks associated with existing waste	Societal	Will the need for such repository be modified?	How should we take into account the possibility of a societal disruption making the availability of a solution more urgent?	low	This uncertainty has limited impacts on safety and decision making and is rated as low.

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THE PROCESS | STEP 1 EXAMPLES

Societal, Political, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.

Societal relates to or involves society. Societal concerns /problems/values Examples of societal questions relevant for the nuclear waste management context would include social acceptance of nuclear waste management, **social media**, and **societal support for education of experts in the nuclear waste management**. Other examples for societal problems that could affect nuclear waste management are **wars, distrust** in institutions, and increase of right- and left-wing extremists.



Screenshots from Youtube.

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THE PROCESS | STEP 1 EXAMPLES

Societal, **Political**, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.

Political means the way power is achieved and used in a country or society. Synonyms: governmental, government, state, parliamentary Examples of political issues: **Changes in parliament** and governmental power, **power politics affecting nuclear waste management, visions and policies regarding nuclear waste management**. Aspects of political issues can also cover issues of future geographical borders of countries.

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
0-2	Waste inventory	Possibility of societal disruptions enhancing the risks associated with existing waste	Political	Are there new, emerging needs associated to changes in energy policy?	Should we consider spent MOX as waste (France)?	Medium	Evolutions in energy policy will surely happen. They will modify, at least, the inventory of the various types of RW, but also sometimes will make new types of wastes appear. Nevertheless, the spectrum of already identified RW allows taking into account some future evolutions, when launching a disposal programme and in the design.

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THE PROCESS | STEP 1 EXAMPLES

Societal, **Political**, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.



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Is waste that is not radioactive still waste or a resource?

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THE PROCESS | STEP 1 EXAMPLES

Societal, **Political**, Technological systems, Financial, Governance and risk governance, Socio-technical and Organizational.



Photo from Wikipedia.

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Yucca mountain is still no repository mostly because of political reasons.

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THE PROCESS | STEP 1 EXAMPLES

Technological systems include technical devices, the organizational routines and procedures, legislative artefacts, and scientific and other **knowledge elements such as skills, rules of thumbs** and norms for handling of the technology.

Societal, Political, **Technological systems**,
Financial, Governance and risk governance,
Socio-technical and Organizational.

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
1a	Robustness of technological choices	Perspective of new technological solutions	Technological	Could our present choices be undermined in the future?	Should we shift to extended interim storage in order to allow transmutation technologies to be used?	Low	Both for the safety and decision-making process, the impact is low. Regarding safety, the repository is designed with the latest knowledge available with a safety margin. New insights and new technologies could change insights, but these insights can still be incorporated in this stage. Regarding the decision-making process, although different techniques might be developed in the future, if a country is already in this stage, they already made the decision to use a repository for their waste rather than wait for new techniques. And therefore, impact is low. As both are low, the research priority is also set to low; it comes close to what if scenarios.

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THE PROCESS | STEP 1 EXAMPLES

Financial is a field that is concerned with the allocation (investments) of assets and liabilities over space and time, often under condition of risk and uncertainty. Examples of financial aspects: The **sufficiency of financial resources regarding** the nuclear waste management and final disposal, the way the financial **resources are collected and the way** sufficient resources are estimated, uncertainties related to management of financial resources, economic depressions.

Societal, Political, Technological systems,
Financial, Governance and risk governance,
Socio-technical and Organizational.

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
1a	Availability of the funds when they will be necessary	Prediction over decades	Financial	Will the necessary funds be available?	How much money should the waste producers allow now for the costs of the planned repository and associated research?	Medium	In this stage, funding problems could lead to delays in the decision-making processes and could impact the safety (longer above ground as not enough research is done or no funds are available for the construction of the repository). But as this is still an early stage, additional funding could be found (extra funding from the state) or collected (waste tariffs) and therefore the impact is only medium for both. As both are medium and the consequence of not having enough funds could be high, research priority is set to medium

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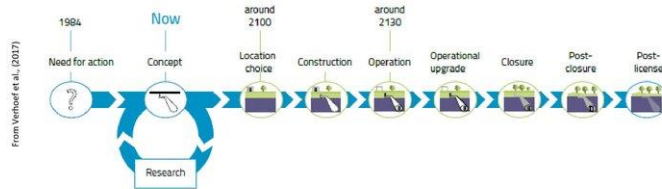
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THE PROCESS | STEP 1 EXAMPLES

Societal, Political, Technological systems, **Financial**, Governance and risk governance, Socio-technical and Organizational.

Financial is a field that is concerned with the allocation (investments) of assets and liabilities over space and time, often under condition of risk and uncertainty. Examples of financial aspects: The **sufficiency of financial resources regarding** the nuclear waste management and final disposal, the way the financial **resources are collected and the way** sufficient resources are estimated, uncertainties related to management of financial resources, economic depressions.



Waste producers pay for the repository but how to make sure that there are enough funds over 100 years?

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THE PROCESS | STEP 1 EXAMPLES

Societal, Political, Technological systems, Financial, **Governance and risk governance**, Socio-technical and Organizational.

Governance comprises both the institutional structure (formal and informal) and the policy process that guides and restrains collective activities of individuals, groups and societies. It aims to avoid, regulate, **reduce or control risk problems** (Renn 2014). Risk governance refers to a complex of coordinating, **steering, and regulatory processes conducted for collective decision-making involving uncertainty** (Renn 2014).

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
1b-2	Compliance wrt repository requirements	Plurality of decision-makers, with various sets of criteria	Governance	How will the points of view of the various actors on the compliance with respect to repository requirements be integrated along the licensing processes?	What kind of obstacles could the licensing processes, other than the nuclear safety, be confronted with?	medium	Some other decision makers can bring other views to decision making process, which can impact the safety and the process itself to some degree. The overall is the uncertainty assessed as medium.

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THE PROCESS | STEP 1 EXAMPLES

Socio-technical systems refer to intrinsic complexity arising from the multidimensional interactions between the human, technical and organizational systems. This is driven by how humans utilize the technologies within the boundaries of the organizational issues, so it is really the interaction of many of the above categories

Societal, Political, Technological systems, Financial, Governance and risk governance, **Socio-technical** and Organizational.

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
1b-2	Availability of useful knowledge	Internal (for the company) uncertainties: managing long-lasting projects	Socio-technical	Will the necessary knowledge be transmitted?	How can the competences be maintained?	medium	If already at the beginning of this project, the management of this project is not good, research that should have been done, might not have been carried out or is not good / conclusive enough. But since this is still in the early stage, additional research can be carried out later in a later stage if the problems and gaps in knowledge are identified. The latter is not a certainty and therefore the impact is medium. This has only a low impact on decision making processes as managing long lasting projects is not part of the decision-making process. At most, it might delay this process and therefore the impact is low. Research priority is set to medium as this is an exceptional long-term process that would require good management.

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THE PROCESS | STEP 1 EXAMPLES

Organizational refers to structures, policies, resources, roles and responsibilities, relationships between the members of an organization, shared values, norms, beliefs and practices in an organization. Examples of organizational aspects: **safety culture, whether adequate resources are allocated to important tasks**, whether organization ensures that there are adequate number of experts.

Societal, Political, Technological systems, Financial, Governance and risk governance, Socio-technical and **Organizational**.

Phase	Object of uncertainty	Identification				Selection	
		Origin of uncertainty		Describing the uncertainty		Priority	Rationale for rating
		Origin	Typology	General question	Examples of specific questions		
4a	Adequacy of activities in operation for the implementation of (operational and long-term) safety provisions	Human and organisational factors	Socio-technical, organizational, Governance	How can changes in organization and safety culture affect the safety of the operation phase?	How is it ensured that knowledge management is taken care of? How is it ensured that people have got adequate training?	High	Human and organizational aspects are inherently involved in the management of operation phase and in any decisions regarding operation, resources and technology. Furthermore, outsourcing of services and work complicates the management of human and organizational aspects. Sociotechnical systems view emphasizes the need for seeing technical and organizational aspects simultaneously (Harvey and Stanton 2014) .

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THE PROCESS | STEP 2 REDUCTION

The result was a long list of uncertainties that are related to the humans. **How to reduce it?**

We used four criteria:

The potential impact on safety.

The potential impact on decision-making.

The existence of referenced work in this field and the interest for further studies and research.

Take the diversity of situations of the national programmes represented by the members of the expert group into account

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THE PROCESS | STEP 2 REDUCTION

Selection: Import! Always record why you have selected (or not) an uncertainty. We did this in the selection column of our graph. This is important as others can better understand your selection criteria.

Q1: A general question related to the uncertainty.

Q2: A more specific question related to the uncertainty. In general these can be regarded as examples.

				Q1	Q2	Selection
				Identification	Who will take part in operation? Will there be an independent body to manage management?	Similar to the safety culture within the company, at national level. Stresses the need for independent oversight.
4.1	4a	Robustness of the safety case via a socio-technical factors	Socio-technical uncertainties related to safety case and political uncertainties related to licensing	Are human and organisational factors taken into account in the safety case? How can political uncertainties affect licensing?	How is efficient communication ensured between the different experts? Who has expertise to have an overall picture of the safety? How do political uncertainties affect licensing? To the possibility of environmental impact properly addressed?	High Socio-technical i.e. interdependencies of societal, organisational and technical aspects create emerging risks and surprises. They play relevant roles in safety case and licensing, and therefore, it is necessary to understand, identify, mitigate and tackle with these socio-technical issues.
4.2	4a	Availability of monitoring results and safety analysis	Socio-technical challenges related to monitoring and long term safety	Will new knowledge, insights or monitoring techniques reveal deficiencies that need to make corrective measures?		High Consequences of weaknesses in monitoring techniques may have safety significance, even though it can be assumed that weaknesses in monitoring techniques will be noticed early enough.
4.3	4a	Adaptivity of activities in operation for the implementation of operational and long-term safety provisions	Human and organisational factors	How can changes in organization and safety culture affect the safety of the operation phase?	How is it ensured that knowledge management is taken care of? How is it ensured that people have got adequate training?	High Human and organizational aspects are inherently involved in the management of operation phase and in any decision regarding operation, resources and technology. Furthermore, outsourcing of services and work complicates the management of human and organizational aspects. Socio-technical systems view emphasizes the need for using technical and organizational aspects simultaneously (Nevry and Martin 2018).
4.4	4a	Robustness of operational safety via a possible social disruption	Social disruptions	How could societal disruptions affect safety?		Low Societal disruption can have severe impacts on operation.
4.5	4a	Robustness of safety via a possible cyber-attacks or	Socio-technical interface, operational (e.g. programming)	How could cyber security and security aspects affect safety?	How should organizations take into account the possibility of intentional actions and events?	High Remits control, programming and related cybersecurity risks and uncertainties are worth of further investigations.

Dumont et al., (2023)

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THE PROCESS | STEP 3 QUESTIONNAIRE

Acronym	WMO	TSO	RE	CS	Near surface	Subs urface	DGD	Belgium	Czech Rep	Finland	France	Germany	Netherlands	Slovakia	Slovenia	Spain	Sweden	Switzerland	Online answers	Word files
V VTT	1	1					1			1									1	
A Andra	1				1	1					1									2
Be BelV/FANC		1			1	1	1													2
Bg BGE	1					2						1								2
Co COVRA/NRG	1					1							1							1
Cv CVR			1		1	1	1		1											3
Ei EIMV		1			1	1	1								1					3
En ENRESA	1				1	2										1				1
G GRS			1										1							2
I IRSN		1			1	1					1									2
N NAGRA	1					1												1		1
O ONDRAF	1				1	1	1													1
Sk SKB	1					1											1			1
Sura SURAO	1					1			1											1
Sur SURO		1			1	1	1			1										3
Sut SUT			1		1	1								1						1
Total	8	5	3	0	8	5	16	2	3	1	2	2	1	1	1	1	1	1	13	14

Lesson learnt: Don't use two ways of getting the data. It will be difficult to combine them. Another issue, specifically at that moment in time, is the large number of questionnaires that were send. People were "questionnaire tired".

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THE PROCESS | THE 10 UNCERTAINTIES

- A: Process for the identification of a workable set of repository requirements
- B: Continuity of the waste management policy along political changes
- C: Robustness of the presently considered safety requirements with regard to the long term
- D: Public acceptance of the repository at potentially suitable or projected location
- E: Schedule to be considered for implementing the different phases of the disposal programme

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THE PROCESS | THE 10 UNCERTAINTIES

- **F:** Robustness of the safety case vis-à-vis sociotechnical factors
- **G:** New knowledge
- **H:** Adequacy of safety-related activities (in siting, design, construction, operation and closure) for the implementation of (operational and long-term) safety provisions
- **I:** Robustness of safety vis-à-vis possible cyber-attacks or programming errors
- **J:** Availability of well-educated human resources, and relevant experts in radioactive waste management along the repository lifetime until closure

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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS

From the sometimes diverging requests of the various actors, what should be the consensus on requirements specification for the design?

How will the set of requirements (regarding long-term safety performance, WAC, environmental impact, etc.) be fixed, taking account the sometimes-conflicting expectations of the various stakeholders?

How could the legitimate requests be included in the RWM policy?

What is the willingness to include requests connected to public acceptance of repository?

How is the consistency of governance policies maintained?

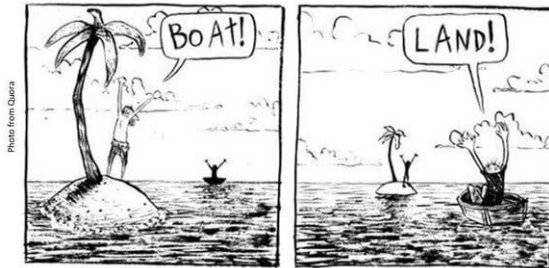
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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS



From the sometimes diverging requests of the various actors, what should be the consensus on requirements specification for the design?

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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS

Potential consequences of uncertainty

Delaying the decisions for a repository may have consequences on the whole schedule of radioactive waste management, when for example waste already exists and is stored in surface facilities without sustainable solution. The absence of definitive solution may also tend to delay the dismantling of the reactors.

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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS

Challenges and potential options for risk management

When developing the approaches to the RWM policy, framework and programme establishment, therefore already at beginning in **phase 0**, it has to be taken into consideration also the **international legal framework** which relates to the different and diverging requests of the various actors which will participate in the RWM activities, and RW disposal establishment.

The requests of other actors may on one **hand conflict** with nuclear **regulatory requirements**, thus cannot be considered, or on the other hand could benefit the process without jeopardizing the safety issues.

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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS



Photo from: <https://tablooo.com/en/press>

A good example of a interaction between the different actors is the Tablooo in Belgium.

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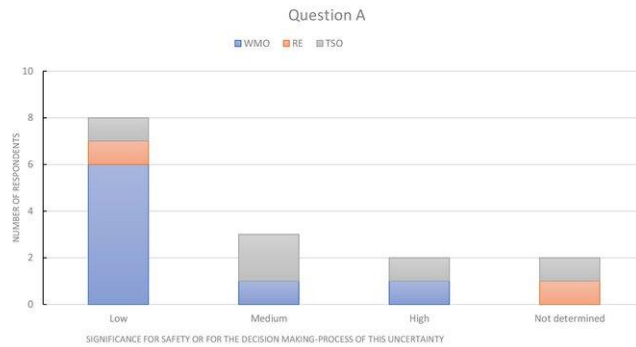
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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS

How did other actors think? Most of them do not consider this uncertainty as having a high significance. Most likely this is because most of them are in a country in which the program is not fully developed.



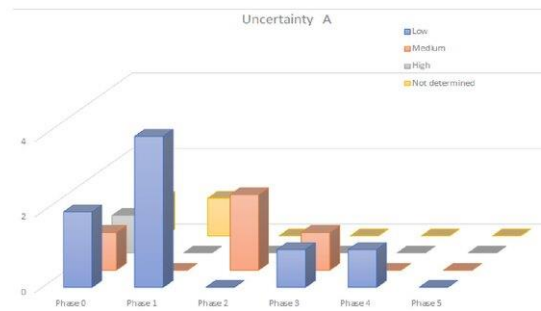
Results from actors on the questionnaire on uncertainty of identification of a workable set of repository regulations for deep geological disposal facility. Note that most actors indicated that the impact on the safety or the decision-making process of this uncertainty is low.

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A | PROCESS FOR THE IDENTIFICATION OF A WORKABLE SET OF REPOSITORY REQUIREMENTS



Results from actors on the questionnaire on uncertainty of identification of a workable set of repository regulations for deep geological disposal facility. From the figure, no clear correlation can be deduced between the phase and the impact of this uncertainty on the safety or the decision-making process.

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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES

Question

Is there assurance of RW policy continuity where political changes can impact the process?

How should the RW disposal process be defined?

Which basic elements should be agreed to assure continuation?

What EU level requirements should be set?

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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES

Potential consequences of uncertainty

This uncertainty, for example, could lead to an **increase in decision time** and potentially in an increase of cost as **longer storage at the surface is needed**. This could also have **consequences for the safety**.

If wastes are still produced when no decision has been made, the **waste may pile up at different nuclear sites** under a **wide variety of conditions** which might have a deteriorating effect on the waste. This in turn could **increase the cost of disposal and might even affect the safety of disposal**.

Could result in the production of waste that **cannot easily be processed or stored**.

Could be a lack of **funds for a repository**.

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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES

Challenges and potential options for risk management

Strong disposal statements at the EU level are helpful. Hence, at the EU level, it would be good to have a more precise description of what are the minimum levels of acceptability, transposing such requirements to the national legal framework, and monitoring the implementation at the EU level (similar system as now adopted by the Waste Directive).

But foremost, **it is important that the RW policy has a (very) broad governmental support** making it more robust to changes in government while keeping the option open to change or reverse the RW policy.

Furthermore, **a participatory approach (i.e., local partnership)** could help to get good local support that could in turn **help to keep the programme on-going.**

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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES

But:

What if the people select a government that is against nuclear energy and as the policy of stopping collecting waste etc? It is the will of the people but will result in problems in the long term.



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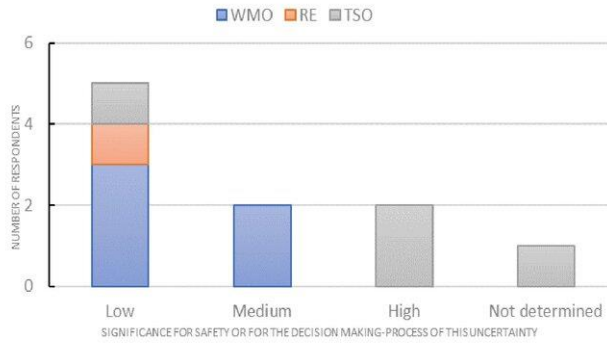
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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES

How did other actors think? Most of them do not consider this uncertainty as having a high significance. This is partly due to the fact that most waste management programs are well established. Once established, it is not easy to stop them.



Results from actors on the questionnaire on continuity of the waste management policy along political changes for deep geological disposal facility. Note that most actors indicated that the impact on the safety or the decision making process of this uncertainty is low.

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B | CONTINUITY OF THE WASTE MANAGEMENT POLICY ALONG POLITICAL CHANGES



Results from actors on the questionnaire on continuity of the waste management policy along political changes for deep geological disposal facility. From the figure, no clear correlation can be deduced between the phase and the impact of this uncertainty on the safety or the decision making process.

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C | UNCERTAINTY ASSOCIATED WITH THE ROBUSTNESS OF THE PRESENTLY CONSIDERED SAFETY REQUIREMENTS WITH REGARD TO THE LONG TERM

Questions

What does the protection of man and the environment mean in the long term?

Which level of effort, supported by present and next generations, should we achieve to protect remote future generations?

What will Man be like in the long term?

Does it make sense to assess safety at 50 000 years based on scenarios that consider human civilization as in the present state?

For long-lived, low-level wastes, which is the suitable trade-off between protection in the long-term (pointing towards deep disposal) and protection against a low level of danger (pointing towards surface disposal)?

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C | UNCERTAINTY ASSOCIATED WITH THE ROBUSTNESS OF THE PRESENTLY CONSIDERED SAFETY REQUIREMENTS WITH REGARD TO THE LONG TERM

What does the long-term protection of Man and the Environment mean?

What level of effort, supported by present and next generations, should we achieve to protect remote future generations?

Potential consequences of uncertainty

A potential consequence of this uncertainty is that it **could increase the time (and costs) to reach** a decision on the concept and associated changes in safety requirements that most agree on.

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C | UNCERTAINTY ASSOCIATED WITH THE ROBUSTNESS OF THE PRESENTLY CONSIDERED SAFETY REQUIREMENTS WITH REGARD TO THE LONG TERM

Challenges and potential options for risk management

The main challenge is related to the evolution of society, **resulting in changing priorities already on the short term (several years to a few decades), and making long-term predictions impossible.**

One potential option for managing the risk of this uncertainty is, for example, defining the adequate **long-term safety goal by involving the whole society.**

Furthermore, the **holistic radioactive waste management programme**, of both surface pre-disposal facilities and eventual repository disposal, **should be consistent with the development plans of the affected region and municipalities.**

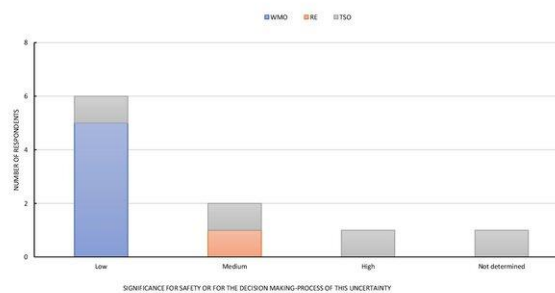
Additional care should be taken to **avoid overlooking some processes** with potentially significant impact, especially when the requirements are formulated at a general level, while safety must be demonstrated at a detailed level.

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C | UNCERTAINTY ASSOCIATED WITH THE ROBUSTNESS OF THE PRESENTLY CONSIDERED SAFETY REQUIREMENTS WITH REGARD TO THE LONG TERM



Results from actors on the robustness of the presently considered safety requirements with regard to the long term for a deep geological disposal facility. Note that most actors indicated that the significance for safety or the decision-making process is low.

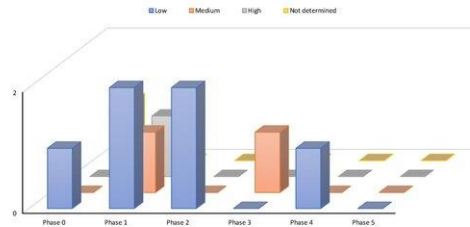
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C | UNCERTAINTY ASSOCIATED WITH THE ROBUSTNESS OF THE PRESENTLY CONSIDERED SAFETY REQUIREMENTS WITH REGARD TO THE LONG TERM



Results from actors on the robustness of the presently considered safety requirements with regard to the long term for deep geological disposal facility. From the figure, no clear correlation can be deduced between the phase and the significance for safety or the decision-making process.

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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

Questions

What is the attitude towards repository in the community?

Has there been any facility for which the public showed the NIMBY effect?

What possibilities for engagement are given to the public?

Are all relevant stakeholders identified and mapped?

Is there a relevant process established to communicate and engage with stakeholders?

How is the communication and stakeholder engagement process integrated into the decision-making process for site selection?

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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

Potential consequences of uncertainty

Site selection can be **delayed for a relatively long period** or, in the most extreme case, it can even be **stopped**. Delaying or stopping the siting can have a **significant financial consequence as the siting process may have to start from scratch again**.

Furthermore, **not building a repository**, due to lack of public acceptance, requires maintaining active measures to ensure the safety and **security of storage facilities that must remain open**. Keeping storage facilities open for extended periods will in the **long term (1) cost even more money, (2) place the burden on future generations and (3) increase the risk associated with the skills loss political instabilities and aging of the storage facilities**. Furthermore, keeping these facilities open increases the exposure times for workers and the number of exposed workers.

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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

Challenges and potential options for risk management

It is **advisable not to start building a repository** if public acceptance is **not high enough**.

Besides local aspects, the **public acceptance of a disposal facility may also be affected by larger economic uncertainties, pandemics, etc.** Thus, any risk assessment should consider relevant threats and possibilities that go **beyond the particular location** but may affect local people's acceptability.

Potential options for risk management of this uncertainty are wide. They will be based on open **communication**, hence access to information, public participation in decision making and access to justice in environmental matters. **It is advisable to have participative decision-making processes implemented at each phase.**

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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

Challenges and potential options for risk management

Specifically, the local community must get strong guarantees that the **negative impact on its environment will be low at short term and in the long term**, and that the **avoid/reduce/compensate strategy** has been properly implemented. It must receive recognition and economic benefits from the efforts they make for the good of the country.

In addition, trust building is important during a **long-lasting continuous dialog** that is independent from future decision points. The **trust building process** requires clear rules, powers and responsibilities for each actor. A potential challenge is that it takes long time to gain trust, but it can be lost within an instant.

Implementing a stepwise process, as **well as popularizing the science** (especially to the new generations, so that they become more familiar with the nuclear industry)



Photo from COVRA.

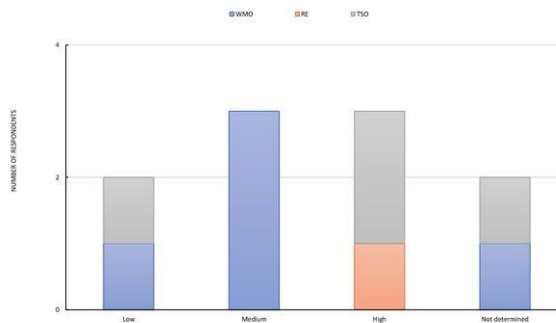
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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

How did other actors think? Most of them think it has some impact, but still relative limited.



RESULTS FROM ACTORS ON THE QUESTIONNAIRE ON THE SIGNIFICANCE OF THE PUBLIC ACCEPTANCE FOR A DEEP GEOLOGICAL DISPOSAL FACILITY. FROM THE FIGURE, IT IS CLEAR THAT THERE NO CONSENSUS ON THE SIGNIFICANCE OF THIS UNCERTAINTY FOR SAFETY OR THE DECISION-MAKING PROCESS.

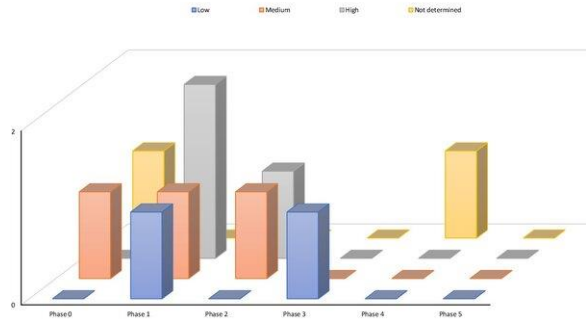
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D | UNCERTAINTY IN THE PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS



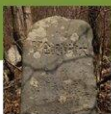

Results from actors on the questionnaire on the significance of public acceptance for deep geological disposal facility. From the figure, no clear correlation can be deduced between the phase and the significance of this uncertainty for safety or the decision-making process. However, most actors indicate that the significance is higher than low.

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UNCERTAINTIES

	Information is available	Information is not available
Level of information is used	(known knowns) 	known unknowns 
Level of information is not used	Unknown knowns 	unknown unknowns 

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SUMMARY AND CONCLUSIONS.

- We have identified 58 human related uncertainties. Although the list might appear to be complete, there is no guarantee it actually is.
- These 58 uncertainties have been characterized and 10 uncertainties were selected.
- These 10 uncertainties have been developed further. This includes the characterization of their impact and potential management options.
- General speaking, actors did not see the 10 uncertainties as being of high significance.

Date



BIBLIOGRAPHY AND/OR REFERENCES.

- Dumont J.-N., Bartol J., Holt E., Zeleznik N. (2023): Deliverable D10.08 “Views of the different actors on the identification, characterization and potential significance of uncertainties related to human aspects”, HORIZON 2020 project EURAD. EC Grant agreement no: 847593.
- Verhoef, E.V., Neeft, E.A.C., Chapman, N., McCombie, C., 2017. Opera safety case.

Date

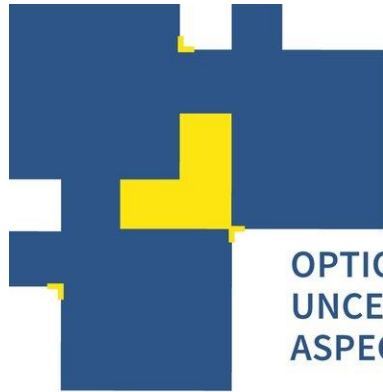
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Appendix L. Slides of lecture C.3

C.3 Options for the management of uncertainties related to human aspects

Prepared by Jitka Miksova, SURO



OPTIONS FOR THE MANAGEMENT OF UNCERTAINTIES RELATED TO HUMAN ASPECTS

Jitka Mikšová, SÚRO

EURAD Training Course on Uncertainty Management
14-16 February 2022, Bel V, Belgium



This project has received funding from the European Union's Horizon 2020 research and innovation programme 2014-2018 under grant agreement N°847593



CONTENT

- Key words
- Learning outcomes
- Introduction to the lecture
- Intro to RWM Programme and aspects of uncertainties
- Programme uncertainties: Public acceptance of the repository at potentially suitable or projected locations
- Programme uncertainties: Schedule to be considered for implementing the different phases of the disposal programme
- Uncertainties associated with initial characteristics: Adequacy of safety-related activities for the implementation of safety provisions (in construction)
- Uncertainties in the evolution of the disposal system and environment: „New knowledge“
- Conclusion
- List of recommended publication and references

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UMAN training materials, C3 - Management options for uncertainties related to the human aspects





KEY WORDS

- RWM Programme implementation
- Human aspects
- Programme uncertainties
- Public acceptance
- Programme schedule
- Repository construction
- New knowledge
- Management actions

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LEARNING OUTCOMES

After the completion of this training lecture, the participants should be able to describe:

- Understand and classify the different types of uncertainties that may need to be managed in a RW disposal programme
- Explain the links between uncertainty management related to chosen human aspects and safety case, the safety case and the decision-making process within programme implementation
- To be aware about global UMAN scheme of uncertainty management strategies
- Explain the main strategies and approaches available to manage uncertainties caused by human aspects
- To have an insight into views of different actors engaged in RWM and involved in EURAD on uncertainty management
- Understand potentially significant uncertainties related to human aspects and discuss their significance
- To have overview about possible main sources of information and recommended publications

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INTRODUCTION TO THE LECTURE

UMAN WP is focused on developing a common understanding among European Waste Management Organisations (WMOs), Technical Support Organisations (TSOs), Research Entities (REs) and Civil Society (CS) of strategies and approaches for managing uncertainties through knowledge sharing and identification of remaining and emerging issues.

The main UMAN goal is to develop a comprehensive overview about different approaches and uncertainty management options to assess and where relevant to reduce risks and optimise safety

Origin of input data/information:

- UMAN Questionnaires disseminated to UMAN participants (WMO, TSO, RE, CS)
- UMAN WSs and Seminars
- UMAN deliverables

The uncertainties' topics identified as of high priority and relevance to safety aspects (safety case) of RW disposal from the addressed respondents:

1. Site and geosphere-related uncertainties (in particular in the course of host rock and site selection processes),
2. **Uncertainties related to human aspects,**
3. Uncertainties related to spent fuel (including those associated with their characterisation, intermediate storage (wet or dry) and possible long-term storage),
4. Uncertainties related to the waste inventory (including uncertainties associated with their pre-disposal management having impact and implications for the safety of the disposal facility).

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CLASSIFICATION SCHEMES USED IN UMAN

- The schemes for classification of uncertainties resulting from UMAN work served as a tool to identify management options depending on the type of uncertainty and temporal aspects.
- The views of different actors on the uncertainties in a safety analysis and in a safety case for geological disposal of radioactive waste and uncertainty evolution have been assessed.

The main goal:

to identify various types of uncertainties potentially relevant to the safety case that the various actors, in particular WMOs, TSOs, REs and civil society, considered as of the highest priority

- The following types of uncertainty were identified :
 - **programme uncertainties,**
 - **uncertainties associated with initial characteristics,**
 - **uncertainties in the evolution of the disposal system and its environment,**
 - uncertainties associated with data, tools and methods used in the safety case, including quality of QA/QC measures,
 - uncertainties associated with the completeness of FEP's considered in the safety case

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GENERIC STRATEGIES AND MANAGEMENT OPTIONS

- **General principles and strategies used for all selected topics:**

- Safety-oriented management processes and principles
- Stepwise and iterative approach
- Regular stakeholder dialog

- **Management of uncertainties:**

- **identification**
- **assessment of the safety relevance** (this can be done either through a preliminary analysis of the safety relevance or through a comprehensive evaluation of the results of the safety assessment)
- **characterisation**
- identification of uncertainties that must be reduced, mitigated or avoided
- classification into “to be reduced”, “to be mitigated”, “to be avoided”
- **representation of safety-relevant uncertainty in safety assessment**
- **Specific actions to (i) reduce, (ii) mitigate or (iii) avoid uncertainty**



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TOPICS SELECTION FOR FURTHER INVESTIGATION

Selection process (more details see in the lecture C2):

Task 3 proposed 10 uncertainties considered as of a **high priority for further investigation**. 4 of them (in red) were subsequently investigated by Subtask 4.2 in more details and served as input to UMAN Subtask 4.3 Workshop 2 (Management options and preferences of different actors regarding uncertainties related to human aspects):

- A: Process for the identification of a workable set of repository requirements,
- B: Continuity of the waste management policy along political changes,
- C: Robustness of the presently considered safety requirements with regard to the long term,
- D: **Public acceptance of the repository at potentially suitable or projected locations,**
- E: **Schedule to be considered for implementing the different phases of the disposal programme,**
- F: Robustness of the safety case vis-à-vis sociotechnical factors,
- G: **Reliability of monitoring results and safety analysis**
- H: **Adequacy of safety-related activities** (in siting, design, **construction**, operation and closure) **for the implementation of safety provisions,**
- I: Robustness of safety performance vis-à-vis possible cyber-attacks or programming errors,
- J: Availability of well-educated human resources and relevant experts in radioactive waste management along the repository lifetime until closure.

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SELECTED TOPICS FURTHER INVESTIGATED IN TASK 4

The uncertainty management options for selected topics presented in this lecture were identified through a compilation, review and synthesis of existing documentation and examples of pitfalls resulting from Task 4 WS 2, taking into account Task 3 results and Task 5/Seminar 3.

Topics selected:

1. Public acceptance of the repository at potentially suitable or projected locations
 2. Schedule to be considered for implementing the different phases of the disposal programme
 3. Adequacy of safety-related activities (in siting, design, construction, operation and closure) for the implementation of safety provisions
 4. Reliability of monitoring results and safety analysis, transferred to „NEW“ knowledge
- associated with programme uncertainties
- associated with uncertainties initial characteristics
- associated with uncertainties in the evolution of the disposal system and its environment

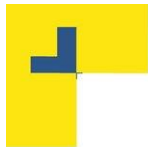
Involved actors: WMO, TSO, REs, CS

Core group members: SURO, BGE, Andra, Nagra

Main goal: to focus specifically on developing a comprehensive overview of different approaches and uncertainty management options to assess, and where relevant, to reduce potential risks and share possible management

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INTRO TO RWM PROGRAMME AND ASPECTS OF UNCERTAINTIES

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RADIOACTIVE WASTE MANAGEMENT PROGRAMME/1

Radioactive Waste Management:

- All administrative and operational activities involved in the handling, pre-treatment, treatment, conditioning, transport, storage and disposal of radioactive waste, (IAEA glossary)
- Activity which refers to the safe treatment, storage and disposal of liquid, solid and gas discharge from nuclear industry operations with the goal of protecting people and the environment (NEA, FSC definition)

Radioactive Waste Management Facility:

Facility specifically designed to handle, treat, condition, store or permanently dispose of radioactive waste (IAEA glossary).

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RADIOACTIVE WASTE MANAGEMENT PROGRAMME/2

Important aspect of every RWM programme:

Disposal facilities for radioactive waste shall be **developed, operated and closed in a series of steps**. Each of these steps shall be supported, as necessary, by **iterative evaluations of the site, of the options for design, construction, operation and management, and of the performance and safety of the disposal system** (Requirement 11: Step-by-step development and evaluation of disposal facilities, IAEA No. SSR-5, 2011).

The long development time for a DGR affords time for the **collection and investigation of further information to enhance the knowledge of repository performance through a carefully planned programme** of testing during the underground construction, emplacement of waste, and any subsequent observational period **until the permanent closure of the repository. At each step, the information is evaluated to ensure the determination that a decision to continue to the next step is appropriately supported** (NEA, 2005).

The uncertainties associated with the evolution of the disposal system must be appropriately considered and managed throughout a repository development programme. At each stage of a stepwise development programme, **decisions should be based on appropriate levels of confidence about the achievability of long-term safety, with the current level of technical confidence established through uncertainty analysis** (NEA, 2005).

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RADIOACTIVE WASTE MANAGEMENT PROGRAMME/3

- The NEA Radioactive Waste Management Committee (RWMC) established the **Forum on Stakeholder Confidence (FSC)** to foster learning about stakeholder dialogue and ways to develop shared confidence, informed consent and acceptance of radioactive waste management solutions. (NEA 2015):
- The FSC experience suggests that, in addition to technical requirements, societal concerns about risk and safety need to be addressed in order for public trust and confidence to develop.
- The FSC recommended that facilities should allow for community oversight and stewardship as a part of enhancing confidence in the DGR process.
- Many countries acceded to the Convention on Access to Information, Public Participation in Decision-Making and Access to Justice in Environmental Matters (**Aarhus Convention**) in order to enhance and respect a **stakeholder involvement** and participation in the RWM programme.

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RADIOACTIVE WASTE MANAGEMENT PROGRAMME/4

Stakeholder by definition:

Stakeholder is any actor – institution, group or individual – with an interest or a role to play in the radioactive waste management process (NEA,FSC)

- Stakeholder involvement **may take the form of sharing information, consulting, dialoguing or deliberating on decisions at different phases**. It has to always be seen as a meaningful part of formulating and implementing good policy.
- It is important **to secure stakeholder involvement through the life cycle of all nuclear facilities**, including spent fuel storage facilities and radioactive waste disposal.
- International experience has shown that, especially in the case of disposal facilities, the **project's progress often relies upon public support**.
- **Decision making on long term SF and RWM is complex**, as it not only concerns the current generation, but also **future** ones, since disposal facilities are designed to operate for many decades and to contain the hazard for thousands of years.
- The **stakeholders' expectations have to be taken into consideration**

The Forum on Stakeholder Confidence Report on Dialogue in the Long-Term Management of Radioactive Waste, 2021, https://www.oecd-nea.org/jcms/pl_56330/the-forum-on-stakeholder-confidence-report-on-dialogue-in-the-long-term-management-of-radioactive-waste

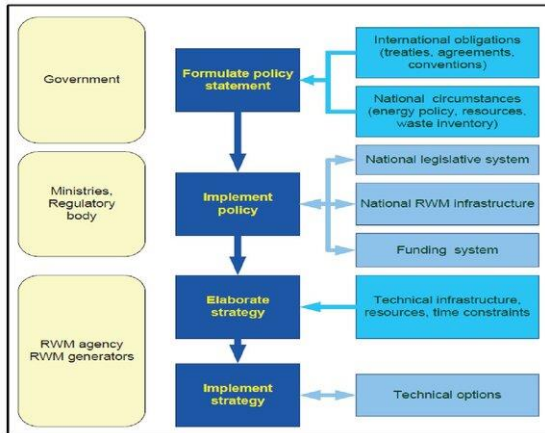
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RADIOACTIVE WASTE MANAGEMENT PROGRAMME/5



- National policy typically addresses the following:
- (a) Responsibilities within the country for spent fuel and radioactive waste management;
 - (b) Arrangements for financing the management (including disposal and decommissioning);
 - (c) Preferred management options for spent fuel, policies for waste disposal, import and export of spent fuel and radioactive waste;
 - (d) Decommissioning of nuclear facilities;
 - (e) Public information and public involvement in related decisions.

Principal steps in the development and implementation of a national radioactive waste management (RWM) policy and strategies, IAEA



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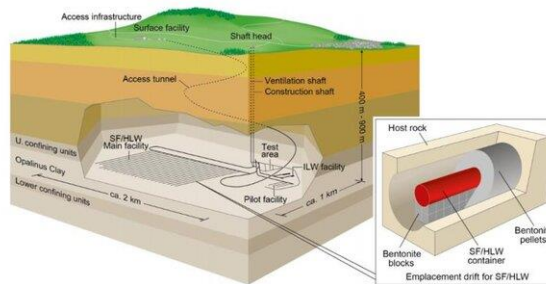
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GEOLOGICAL DISPOSAL FACILITY CONCEPT SELECTION

Selection of disposal concept depends on:

1. National policy
2. **Implementation strategy (considering e.g. timescale, novel technologies development)**
3. Legal and regulatory framework
4. National inventory (volume, characteristic, location, etc)
5. Scope of nuclear activities (past, running, envisaged)
6. Funding, cost estimate
7. **Existing infrastructure (technical, human)**
8. Geological background, host rock suitability
9. **Conflict of interest (land use and nature protection, transport networks, etc.)**
10. **Stakeholders influence and expectation**
11. **Public acceptance**
12. **Political commitment, preferences**



Source: "Swiss Geological Studies to Support Implementation of Repository Projects: Status 2015 and Outlook", Nagra, 2015



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SUCCESSFUL FACTORS FOR SNF/RWM PROGRAMME IMPLEMENTATION

- Long term political commitment to resolve SNF and RW issues
- **National strategy and discipline policy**
- Well defined liabilities and roles, governance
- Early established funding system
- Early involvement of regulatory body in project and strategy planning
- **Agreement of local community in hosting repository and in stepwise licencing process**
- Implementor programme establishment (structured, stepwise, open and defendable using graded approach and rolling document strategy)
- Good safety culture and importance of dialog between regulatory body and implementer based on comparable levels of technical competence
- Transparency and engagement of public (supported by domestic and international scientists and technical experts)
- **Knowledge management, knowledge transfer, education and training, human resources sustainability**

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UNCERTAINTY ASSOCIATED WITH THE NATIONAL RWM PROGRAMME AND OTHER PREVAILING CIRCUMSTANCES

Uncertainty associated with the national RWM programme and other prevailing circumstances:

include, in particular, **societal and political uncertainties** relating to the waste to be disposed of, stakeholder conditions, the regulatory framework, the disposal concepts and schedule, available financial resources, and available skills and experience.

Two types of uncertainties have been selected by UMAN for further investigation for the category “programme uncertainties”:

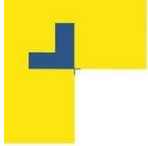
1. **Public acceptance of the repository at potentially suitable or projected locations**
2. **Schedule to be considered for implementing the different phases of the disposal programme**

Both types of these uncertainties are closely interlinked and overlap

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PROGRAMME UNCERTAINTIES

TOPIC 1: PUBLIC ACCEPTANCE OF THE REPOSITORY AT POTENTIALLY SUITABLE OR PROJECTED LOCATIONS

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PUBLIC ACCEPTANCE UNCERTAINTIES - GENERAL REMARKS



- This uncertainty comes into play mostly during site selection
- There is no correlation between the impact of this uncertainty and the phase of the programme
- Regarding the evolution of this uncertainty, there is a consensus that it will decrease over time, especially when a participative decision process is implemented at each phase to ensure that the decisions made at each phase are supported by the public
- Since this has no direct impact on safety, this uncertainty was assessed as having a medium level of significance for safety
- It may not be strictly limited to the local community hosting the repository, as other communities in the repository vicinity may feel also concerned, especially because of the transportation of the waste to the repository, and lower benefits expected (in terms of jobs, infrastructure, etc.).
- In the early programme phases, this uncertainty may lead to a total failure of a siting process
- it is difficult to decrease this uncertainty through RD&D.
- the stakeholders may require new elements to be included in the decision-making process (more discussions, more meetings, new ideas, etc.), which will impact the decision-making process and potentially the schedule
- associated uncertainties concern the measurement of the public acceptance, in particular how to determine that a sufficient level of acceptance is reached and how to measure the acceptance level.

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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE /1

Identification:

- In the framework of a **decision-making process**
- Should be identified during the **early stages** of disposal programme implementation: programme initiation, siting and characterization.
- Through **open communication** between different stakeholders
- Through an active **public participation**
- Through an independent **safety reviews**
- High importance within **siting**



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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE /2

Safety relevance:

- **No direct impact on safety**
- If site selection is also driven by **public acceptance criteria**, the safety concepts and finally disposal facility safety may depend **indirectly** on public acceptance.
- **It can cause important delays in the implementation** of the disposal programme which may eventually lead to safety issues in storage facilities or during transportation
- the safety relevance is very much dependent on resulting delays in programme implementation, having impact on lifetime and capacity of storage facilities, and on possible ageing phenomena affecting the waste and these facilities.
- Different stakeholders may introduce **new safety-related issues** that need to be considered or reassessed in the safety analysis.

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CHARACTERIZATION

- The **stakeholder acceptance becomes an increasingly important issue** in a number of national programmes (e.g. the existence of a requirement that the siting of the facility does not take place without the consent of the public concerned).
- Should be done during the early stages of disposal programme implementation: programme initiation, site selection and characterization.
- Possible approaches to characterize these uncertainties include:
 - **Multi-channel communication** with stakeholders: public meetings, newspapers, door to door, webinars, web site, etc.
 - **Identification of stakeholder categories** as well as of their interest, influence and relationships through stakeholder analysis.
 - Quantitative representation using statistical methods on data, consideration of accuracy of measurements, e.g. socio-economic studies.
- These uncertainties are closely related to the consideration of the expectations of different stakeholders notably on various operational issues such as operational monitoring, the retrievability of the waste, costs, conditioning of radioactive waste, location of surface installations etc.

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CLASSIFICATION AND ASSOCIATED ACTIONS /1

This uncertainties **cannot be avoided** but **could be adequately reduced or mitigated**

In particular, achieving a level of sufficient public acceptance is needed.

The following measures can be taken to reduce this uncertainty:

- Development and establishment of a **fair, sustainable, transparent procedure for the repository site selection**
- **Mutual dialogue between stakeholders**, including the public, and the timely involvement of the public in the decision-making process. It should be a continuous dialog with the public instead of one-way communication (i.e. issuing of information), particularly with a network of citizen experts.
- Communication and **stakeholders engagement strategy/concepts** are a tool for managing the acceptance uncertainty **but may be unsuccessful**.
- Development of **support programmes in preselected sites**.
- Performance of **RD&D activities** with a focus on transdisciplinary research and social science.

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CLASSIFICATION AND ASSOCIATED ACTIONS /2

The following actions can be taken to reduce this uncertainty (cont.):

- Ensuring **mutual understanding** of the repository safety during the operational phase and in the long-term (timespan of one million years) and on possible site/repository evolutions.
- Public acceptance may also be affected to some extent by the way in which **conflicts of interest** (including the interests of the public concerned) are resolved, and the degree of compatibility of the disposal project with the site development plan.
- **Building and/or improving credibility** of the responsible organizations/institutions involved in the management of radioactive waste and decision-making process (e.g. clearly defined functions and responsibilities, independency, transparency, learning organizations, self-questioning process, science, open to criticism).
- **The role of the various reviewers**, namely the regulator and its TSO, is prominent to foster confidence on safety.
- **Independent safety reviews by the regulator and its TSO** and/or by other independent experts mandated by civil society organisations.

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REPRESENTATION IN SAFETY ASSESSMENTS

Even there is no direct impact on disposal facility safety it may be appropriate to assess the possible implications for safety of this uncertainty in the safety case. In particular, **possible consequences of the delays resulting from a lack of public acceptance could be assessed in the safety assessment**.

If certain requirements related to the repository are set due to **acceptability constraints** (e.g. reversibility), the **consequences of the resulting provisions on safety shall be presented in the safety assessments**, e.g. the measures for retrievability **need to be shown not to impact post-closure safety** as requested in one of the national regulation.

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PUBLIC ACCEPTANCE - ADDITIONAL REMARKS

- **NIMBY effect**, extending licensing process, veto right, prospects of new technologies.
- From facility construction phase, it should be demonstrated that any **remaining uncertainty will not jeopardize the safety of the facility**.
- The uncertainty is indirectly addressed e.g. in **the Council Directive 2011/70 /Euratom - basic requirements for ensuring the transparency of the process with the active involvement of the municipalities and the public concerned**. These requirements are usually implemented also into **national legislation**.
- The **advances in science and technology** over future decades may allow **development of novel concepts** that not only meet stakeholder wishes but also provide benefits in terms of both pre-and post-closure safety



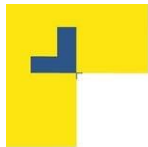
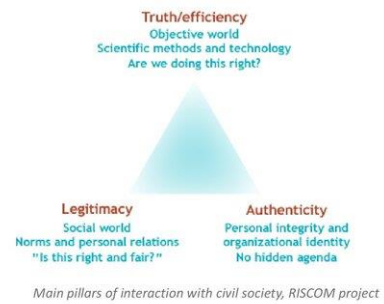
•To avoid negative “surprises” in the process from both sides, future disposal facility implementer and public

Uncertainties should be described in a way that **stakeholders can understand what we do not know and why we think this is not relevant or why it is relevant for safety** and, if so, **how and when we will deal with it**.

NIMBY, an acronym for “Not In My Backyard,” describes the phenomenon in which residents of a neighborhood designate a new development (e.g. shelter, affordable housing, group home) or change in occupancy of an existing development as inappropriate or unwanted for their local area, (wiki)

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PROGRAMME UNCERTAINTIES

TOPIC 2: SCHEDULE TO BE CONSIDERED FOR IMPLEMENTING THE DIFFERENT PHASES OF THE DISPOSAL PROGRAMME

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PROGRAMME UNCERTAINTIES – SCHEDULE – GENERAL REMARKS

This uncertainty is related to **possible delays in the implementation of the programme schedule**



This uncertainty can occur **in all phases of disposal facility lifecycle until its closure.**

The schedule may be affected by uncertainties regarding insufficient support and acceptance by the public (or by other stakeholders), provision of sufficient financial and raw material resources, human resources, and the availability of appropriate technologies, by accidents during repository construction and operation and also due to political changes, etc.

Some of these aspects being subjects to **quick changes**, thus these uncertainties are **very difficult to characterize.**

It affects the timing of the agreement on the completeness of the safety case and resources for evaluation.

Technical, administrative and social problems may significantly change any implementation plan.

Implementation process should be checked regularly, including at each milestone, for updating the planning.

It is expected, according to the preliminary governance of the project, that the **major decision points will be subject to public consultation.**

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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE



Identification:

- should start in the phase of **policy framework and programme establishment** and be pursued during subsequent phases (Site evaluation and site selection, Site characterization, Facility construction)
- should be identified **through risk analysis** (e.g. risk matrix, risk register, hazard identification, scenario analysis) and project management.
- can be performed through a **stepwise approach and open communication between the different stakeholders**

Safety relevance:

- the impact of this uncertainty on safety can be **indirect and/or direct** - depending on the phase of the programme. No direct consequences are expected before construction of the facility. Indirect consequences are possible during all phases and remain until repository closure.
- Uncertainties on the date of availability of the disposal facility may lead to necessary decisions for **extending the duration of interim storage**. Delays can also lead to a diminished robustness vis-à-vis potential societal disruptions, to **enhanced ageing of waste packages** before final disposal and affect the **ability to retrieve the waste** safely if necessary.
- In the case of DGR, delay of installation of stabilizing measures may cause **stability problems and may affect long-term safety as well**. It is possible to assess the potential consequences of this uncertainty through e.g. mining experience, numerical calculations considering stability of mining excavations and barrier integrity, etc..
- Delays can cause loss of expertise and knowledge, which can also have an indirect effect on the safety.
- Further, delays in the programme implementation can cause **increasing costs**, which in turn **may affect the quality of the execution of future activities due to lack of resources and can thus adversely affect the repository safety**

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CHARACTERIZATION

- This uncertainty is **very difficult to characterize and quantify**, it is directly related to a number of other “programme uncertainties”
- **Should be characterized during the early stages** of disposal development – program development and subsequent **revision during the next phases**: site selection, characterization...
- Should be characterized **via stepwise approach, with intense communication with the stakeholders at each step**; draft planning of the successive major decisions updated and shared regularly.
- The actual schedule is the **result of a mixture of technical constraints and societal aspects** (e.g. of strategies of the various actors, with sometimes opposite interests), leading to unreducible uncertainties.
- It could be derived through **the determination of the degree of public acceptance** in the frame of **socio-economic studies** (e.g. NIMBY, it depends on communication with public and level of active stakeholder engagement, on **flexibility** of the disposal facility development programme)
- The main question for this uncertainty is **the appropriate timing**, for involved decision makers especially, with regard to the **agreement on the completeness of the safety case and resources for evaluation**.

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CLASSIFICATION AND ASSOCIATED ACTIONS /1

- **This uncertainty needs to be reduced and mitigated but cannot be avoided.**
- This uncertainty will presumably be reduced with the implementation of the disposal programme throughout the phases (stepwise approach).
- The following actions can be undertaken **to reduce this uncertainty**:
 - through the implementation of **good project and risk management practices**.
 - Through a **more robust planning** by establishing a draft planning of the successive major decisions regarding the repository in the (pre-)licensing phase and beyond, and presenting it for comments to the various stakeholders, so that a **consolidated shared view on the planning** may emerge.
 - **Holistic approach** for the waste management programme with soft links between components.
 - Professional, forward-looking **human resources planning**.

We must expect significant delays in the schedule as the normal evolution of the project, thus it requires managers and scientists to accept it as an irreducible uncertainty and to plan suitable mitigation measures. Past experiences have shown sufficient evidence of schedule changes.

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CLASSIFICATION AND ASSOCIATED ACTIONS /2

- The following measures allowing to **mitigate** this uncertainty:
 - Implementation of a **participative and transparent decision-making process at each programme phase** in order to ensure that the decisions made at each phase are supported by the various stakeholders
 - **Planning the necessary resources** (not only financial) to complete corrective action in connection with delay.
 - Flexibility measures:
 - The provision of a **flexible disposal programme implementation**.
 - **Repository design that will be flexible and adaptable**, i.e. **robust vis-à-vis modifications of the waste emplacement schedule robustness of the design with respect to ageing processes**
 - **Defining a waste management programme (interim storage + repository) that will be flexible**, i.e. where the licensing decisions for the repository cannot be suspected of being dictated by the fact that the storage capacity is saturated.
 - In the construction phase and the operation and closure phase of the programme, the potential consequences of this **uncertainty on the stability of the disposal facility can be handled by observation methods** (monitoring, e.g. deformations of mining cavities), can be quantitatively described
 - Different engineering methods and assessed by numerical calculation should be applied.

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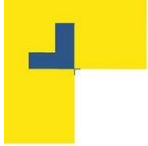
REPRESENTATION IN SAFETY ASSESSMENTS

- This can be **done by defining scenarios investigating the potential impact on safety** of delayed backfilling or closure.
- The consequences of this **uncertainty for the stability of the facility** may also be explicitly addressed, in the licencing procedure by the regulatory authority in charge of underground safety aspects.

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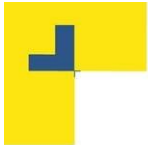


End of the part 1

Thank you for your attention!

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UNCERTAINTIES ASSOCIATED WITH INITIAL CHARACTERISTICS

ADEQUACY OF SAFETY-RELATED ACTIVITIES FOR THE IMPLEMENTATION OF SAFETY PROVISIONS (IN CONSTRUCTION)

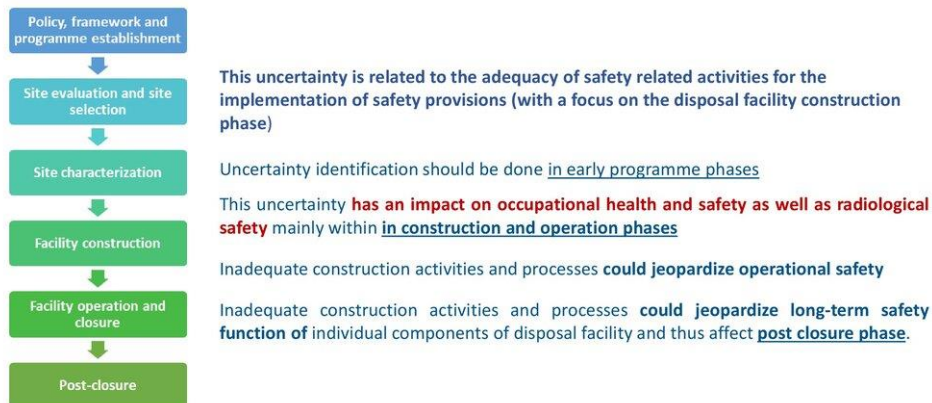
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UNCERTAINTIES ASSOCIATED WITH INITIAL CHARACTERISTICS – GENERAL REMARKS



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UNCERTAINTIES ASSOCIATED WITH INITIAL CHARACTERISTICS

From CS view: It is uncertainty generated by human activities during the implementation of disposal facility

Question - What is a gap between theory and safety case VS concrete implementation?

From WMO view:

How can it be ensured that the safety provisions taken into account in the safety assessment are adequately implemented?

How robust is the safety assessment vis-à-vis any potential inadequate implementation of the safety provisions?

Key uncertainties:

Long-term safety: Inadequate safety-related activities in the construction phase potentially affect long-term safety functions of individual components and as a consequence may affect the performance of the whole system in the long-term.

Operational safety: Inadequate safety-related activities in the construction phase may cause local instabilities (roof falls, collapse of drift face) with consequences to conventional and radiological safety in the operational phase.

IAEA GEOSAF II project - Managing integration of pre-closure and post-closure safety in the Safety Case for geological disposal:

„Development of a process to assure that construction and operation of a geological disposal facility will deliver the expected post-closure safety functions.“

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ADEQUACY OF SAFETY-RELATED ACTIVITIES FOR THE IMPLEMENTATION OF SAFETY PROVISIONS IN CONSTRUCTION - INTRODUCCION

This uncertainty :

- is linked to safety provisions in a disposal facility construction phase, e.g. construction errors
- has an impact on the properties of the components of the disposal system in the real state.
- **includes the socio-technical aspects associated** with the activities required for the implementation of the safety provisions
- It is affected by changes in organisation and safety culture, lack of knowledge management, inadequate training of the staff

Existing construction codes and operating rules take into account operational and long-term safety, BUT:

- **Rules are subject to interpretation.** Implicit requirements, evident at the time the rules are set, may be ignored later due to lack of appropriate knowledge management.
- **Rules may be violated by ignorance, laziness, greed or malice of employees.** The quality insurance system in place may allow violations to go undetected.
- **Interactions between humans and technology** (e.g. human-machine interface) **could also contribute to the uncertainties occurrence.**

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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE/1

This kind of uncertainties and their safety significance could appear over **construction and operation**, including closure phases of the disposal facility and **should be reduced and/or mitigated** through the implementation of suitable construction activities.

- Uncertainties on the proper implementation of safety-related activities during repository construction **need to be identified at early programme phases** before facility construction phase.

Identification through:

- lessons learned and **available experience from similar construction activities** (in other disposal facilities or in other fields such as mining and tunnelling), can be very useful in identifying possible construction errors and assessing their impact
- **monitoring system** during repository construction, operation and closure, (i.e. in case the behaviour of the repository system differs from expectation)
- **inspections** of accessible parts of the facility.

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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE/2

- impact **on occupational health and safety + radiological safety** in the construction and operational phases (i.e. operational safety).
- inadequate construction activities and processes **could jeopardize long-term safety functions of individual components of the disposal system.**
- **oversight of several design steps by licensing authorities**, having expertise in different relevant fields (safety assessment, mining, civil engineering, etc.):
 - Deep scrutinising of safety case by the regulator and its TSO,
 - External audits or inspections by the regulator and its TSO.
- **framework of Periodic Safety Review (PSR), which evaluates separately “human factors”**

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CHARACTERIZATION

- This uncertainty is related to the management provisions for the activities related to safety during repository construction.
- It is affected by:
 - changes in organization and safety culture
 - lack of knowledge management, and inadequate training
 - lack of experts involved in construction activities

Can be characterised through:

- the quantitative description of the uncertainties may be limited, however, their quantification, respectively **quantification of their impact**, might be needed when their representation in the safety assessment is necessary.
- a **certain percentage of undetected defects may be quantified based on return on experience** from construction activities of other disposal facilities or from other similar activities

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CLASSIFICATION AND ASSOCIATED ACTIONS /1

These uncertainties need to be reduced and where possible avoided or mitigated.

The following actions can be undertaken to reduce this uncertainty:

- the disposal of radioactive waste should be treated from the socio-technical perspective (and not as a solely technical issue),
- interactions between human, technology and the organization is recommended/needed
- appropriate design of a disposal facility according to current national and international standards needs to be considered in order to reduce future construction errors or uncertainties

Basic means to reduce/mitigate this uncertainty:

- implementation of a quality assurance system within the organization (quality assurance documents, training, dedicated teams of quality experts, independent levels of quality management and control within the organization, etc.)

The quality assurance system ensures that the parameters stay within the domains taken into account in the safety assessments

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CLASSIFICATION AND ASSOCIATED ACTIONS /2

Uncertainty reduction, avoidance or mitigation can be also achieved through:

- inspections during or following the repository construction of the proper implementation of procedures as well as of as-built properties
- external audits already by the regulator and/or its TSO, and by quality certification auditors during the design phase on the repository project as well as on construction activities
- set of behavioural expectations and fostering a strong safety culture (Safety Culture Development Programme) among the employees involved in construction activities (including subcontractors) and repository operation
- implementation of a feedback programme is essential to reduce the uncertainties and the risks associated.
- consideration of processes from other repositories already in operation (at least for L/ILW) in order to benefit from available experience and expertise.

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CLASSIFICATION AND ASSOCIATED ACTIONS /3

Uncertainty reduction, avoidance or mitigation can be also achieved through (cont.):

- **ensuring compliance** of the disposal **facility design and construction methods with the current state of science and technology, international experience**, experience from the operation of nuclear installations with regard to ageing of its systems, structures and components throughout life cycle of the disposal facility.
- **regular updates and implementation of a national RD&D programme** in accordance with the disposal programme schedule, including demonstration tests
- documentation and knowledge management
- networking and collaboration with external groups
- education of specialists in fields related to RW disposal
- **good governance at all!**

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REPRESENTATION IN SAFETY ASSESSMENTS/1

- Human factor is a separately evaluated element **within Periodic Safety Review (PSR)** thus it is necessary to establish the safety case iteratively, in several steps (feasibility, safety options, preliminary safety report, etc.) over a long period of time over a long period of time allowing for extensive reviews by the regulator and its TSO.
- **The safety case shall specify how uncertainties are identified, how they are characterized and what is the approach for their management**
- The safety assessment must **address the technical feasibility** of the proposed repository design options to demonstrate that as-built properties are as assumed in the safety case. This implies use of proven methods/techniques, in which the following is stated: **“The licensee shall construct the disposal facility in accordance with the design as described in the safety case and by application of appropriately proven techniques”.**
- These **new techniques need future confirmation through experimental tests**: demonstrated through qualification programme defining systematic approach to the qualification of new technology, ensuring that the technology functions reliably within specified limits.
- The approach is applicable for components, equipment and assemblies, which can be defined as new technology

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REPRESENTATION IN SAFETY ASSESSMENTS/2

Question/challenge arisen:

- how to reconcile the goal of **applying advanced technologies and/or the latest scientific results** ?
- how to comply with the **application of the best proven methodologies and technologies (BAT principle)** ?

Therefore a choice has to be made **between the uncertainties associated with this latest scientific and technological progress and the robustness principle and/or to find a balance between such progress uncertainties and SAHARA principle (Safety As High As Reasonably Achievable)**.

- With respect to this, **Technology Readiness Level (TRL)** has to be taken into account (e.g. **two technologies with different levels of maturity cannot be compared**) and also its potential affect on a possible optimization process has to be considered.
- Optimization should be kept reasonable and efficient

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REPRESENTATION IN SAFETY ASSESSMENTS/3

- Implementation of the repository **“Defence in Depth”** principle (DiD)
- DiD is one of the **fundamental safety principles identified by IAEA, it includes protective measures** against disturbing events and processes, presence of aboveground constructions, controls and corrective measures to prevent defects and damages to the disposal (developing quality control measures during construction based on clearly defined conformity criteria/quality requirements and periodic testing and inspections), maintaining the performance of system components when subjected to construction errors (e.g. through provision of safety margins in the design, diversity and redundancy, multi-barrier system, etc.).
- **Periodic safety reviews along the disposal programme, knowledge management system, quality management system, collaborations with universities are considered as helpful tools.**
- **Changes of personnel along the long-term disposal programme make documentation and knowledge management highly important.** Therefore, it is necessary to **have an adequate knowledge management already in place at a very early programme stage.** This will help to improve safety for the repository.

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REPRESENTATION IN SAFETY ASSESSMENTS/4

Possible constructions errors can be represented explicitly in the safety assessment:

- through using of specific parameter values or
- through consideration of developed scenarios.

This might be done in the framework of (i) the expected repository system evolution or (ii) reference scenarios or (iii) dedicated altered evolution scenarios, depending on the probability of occurrence of the deviations.

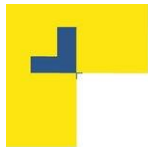
The **quality assurance system** aims at guaranteeing that the parameters stay within the domains taken into account in the safety assessments

Data on the probability of these errors occurring are relatively rare and usually available on a limited scale. This can also be done through conservative assumptions/parameter values or specific scenarios and by including a “poor quality construction” FEP-category in the FEP database.

Safety case needs to demonstrate not only “a good system” but also “good processes”, namely a “good safety analysis”, i.e. it needs to prove that these uncertainties have been dealt with adequately (e.g. QA system, clear roles, etc.).

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UNCERTAINTIES IN THE EVOLUTION OF THE DISPOSAL SYSTEM AND ENVIRONMENT

„NEW KNOWLEDGE“

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UNCERTAINTIES IN THE EVOLUTION OF THE DISPOSAL SYSTEM AND ENVIRONMENT - „NEW KNOWLEDGE“

This uncertainty related to a “new knowledge” is associated with the evolution of the disposal system and environment (e.g. ignoring the possible magnitudes of disturbing events, unexpected new knowledge)

“New knowledge” means here that the knowledge has emerged by research and monitoring, but also is new for certain actors and has been ignored by others.

- Relevant to **all phases of disposal facility lifecycle**
- Includes any new knowledge becoming available in the course of a RWM
- Occurrence of unexpected new knowledge or findings emerged by research, characterization or monitoring (i.e. **unknown unknowns**),
- the knowledge is available but is new for certain actors which were not aware of this knowledge (i.e. **unknown knowns**)
- the knowledge is available but is ignored by certain actors (i.e. **ignored knowns**). Ignoring the possible magnitudes of disturbing events (e.g. in Fukushima, the tsunami was not part of the key FEP) .
- International / broad exchange helps to identify and rank/assess “new” knowledge.

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UNCERTAINTIES IN THE EVOLUTION OF THE DISPOSAL SYSTEM AND ENVIRONMENT

<i>Knowledge is available</i>	<i>Lack of knowledge</i>
Known Knowns <i>What is known & used</i>	Known Unknowns <i>What we know we don't know</i>
Unknown/Ignored Knowns <i>What is known but we are not aware of or do not consider</i>	Unknown Unknowns <i>What we don't know we don't know</i>

Uncertainties associated with FEP completeness in relation to the “information awareness” classification.

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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE/1

- **Unknown unknowns, by definition, cannot be identified**
- Ignored knowns can be identified, e.g. through audits and independent reviews
- Unknown knowns can be identified through:
 - steady monitoring of the research landscape and research developments (new knowledge/technologies, scientific findings, etc.),
 - international/national exchange with experts from the field of radioactive waste management,
 - interdisciplinary exchange of knowledge and experiences,
 - consideration of pitfalls and lessons learnt.
- **The identification of unknown knowns leads automatically to their elimination.**
- Known unknowns can be identified through:
 - exploration activities,
 - operation of a monitoring system.
- **The long-term evolution of the disposal system – new knowledge could lead to a revision of the safety assessment and the repository design**

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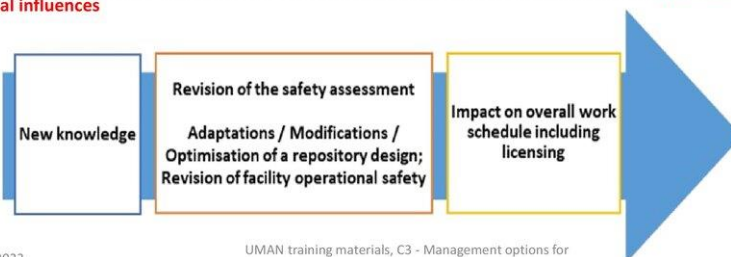
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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE/2

Safety significance over time **expected to increase at the beginning of the site selection process:**

- from low/medium in Phase of Policy, framework and programme establishment
- to high in Phases Site evaluation and site selection; Site characterisation; Facility construction; Facility operation and closure,
- and high even in Post-closure phase in connection with post-closure issues such as post-closure monitoring and possibly radioactive waste recoverability
- One mean of assessing the safety relevance of this uncertainty e.g. during siting could be **systematic analyses of its potential influences**



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IDENTIFICATION AND ASSESSING THE SAFETY RELEVANCE/3

- In case of a DGR, the most “unknown known” knowledge is the close connection between repository safety and mining safety (at least in the operational phase). The death **risk associated with mining activities is much higher than the radiological risk**. This leads to different risk perception and acceptance as well as different safety cultures among different groups of staff.
- The safety significance of „unknown and ignored knows“ is **considered as high as the return on experience shows that past accidents/incidents** were often associated with these uncertainties. An example of that would be the Fukushima NPP accident due to the earthquake and subsequent tsunami and ignorance of possible magnitudes of these disturbing events.
- The safety significance of „unknown and ignored knows“ will remain **potentially important** during the different programme phases (i.e. the consequences could only appear much later).
- The uncertainty management strategy should ensure that, **at the end of the decision-making process, no remaining uncertainty can potentially jeopardize repository safety**.
- **Question arisen:** how **new results**, emerging in the course of the implementation of a disposal programme (e.g. from monitoring), **can be taken into account in the safety case and the safety assessment**.

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“NEW KNOWLEDGE” UNCERTAINTY - CHARACTERIZATION

Characterization of these uncertainties can be realized through:

- **Stepwise approach:**



Safety case established in several successive steps, prior to the licence for construction, each of them being reviewed in detail by the regulator and its TSO

The regulator will authorize only operations that will have to be duly demonstrated as safe

- Methods have to be defensible with respect to **good practice and quality requirements**
- Assessing the relevance of the new knowledge in the site selection procedure - **systematic analyses of potential influences**
- **Safety margins:** values of the parameters taken into account are set in order to cover uncertainties related to known unknowns.
- Intense **communication with the stakeholders** and interdisciplinary cooperation at each step

The categorization into known unknowns, unknown/ignored knows, and unknown unknowns constitutes a way to characterize these uncertainties in the best manners.

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CLASSIFICATION AND ASSOCIATED ACTIONS/1

This uncertainty is difficult or impossible to be reduced as any uncertainty about the occurrence of unexpected new findings or knowledge is representing a **persistent uncertainty**.

BUT: safety-relevant unknown unknowns need to be avoided or reduced as much as possible, this can be done by carrying out the following actions:

- **Well planned RD&D programme** in the area of RAW/NSF management
- **Site selection:** determination of criteria to properly characterize the site and the host rock, refined modelling: 3D modelling can also raise some new issues/new knowledge
- **Data acquisition and site characterization, monitoring** (it goes beyond the individual objectives of operational safety and compliance with the set values and is also aimed at gaining knowledge).
- **Appropriate design & construction** of a disposal facility (e.g. use of proven methods and materials) with respect to **good practice and quality requirements**.
- **Establishment of limits & conditions/waste acceptance criteria**
- **Interactions with stakeholders.**
- **Exchange with international experts**



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CLASSIFICATION AND ASSOCIATED ACTIONS/2

AND: Safety-relevant unknowns and ignored knowns (e.g. deliberately ignoring the possible magnitudes of disturbing events) **need to be minimized**, this can be done by carrying out the following actions:



- ✓ **Implementation of a sound management system** ensuring notably an adequate performance of the uncertainty management process (e.g. supported by uncertainties roadmap).
- ✓ **Knowledge management** (including training and qualification of the staff involved in safety-relevant activities); **development and application of standardized approaches for managing information, data and knowledge***
- ✓ **Systematic FEP management** - derivation of national and international FEP lists and database and their regular review/updating.
- ✓ **Peer review of scientific developments**
- ✓ **Monitoring** - beyond the individual objectives of operational safety and compliance with the set values and is also aimed at gaining knowledge.



* E.g. NEAs RWMC: Working Party on Information, Data and Knowledge Management - WP-IDKM. (January 2020 - December 2022). Subjects for the working party include safety issues, knowledge management, archiving and preservation of data and information. It strives to propose standardized approaches for managing information and data of radioactive waste and repositories.

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CLASSIFICATION AND ASSOCIATED ACTIONS/3

AND: Safety-relevant **unknowns and ignored knowns** (e.g. deliberately ignoring the possible magnitudes of disturbing events) **need to be minimized**, this can be done by carrying out the following actions:

- ✓ **Self-questioning and learning associated** with the reversible character of the site selection procedure (for stepwise regulatory and/or policy strategies to be credible): decisions must be reversible or at least modifiable in view of new information, to the extent that this is feasible. Therefore, reversibility refers to the possibility of reconsideration of one or a series of steps at various stages of a programme. This involves a review of earlier decisions with the appropriate stakeholders and requires that the necessary means to reverse a step be available. The reversibility within a planned process should probably be discussed ahead of time.
- ✓ **Implementation of an experience feedback programme** → to take benefit of the experience gained in other facilities
- ✓ **Safety culture** – e.g. implementation of a Safety Culture Development Program → improvement of the responsibility (and in particular of the safety leadership), professional competence and credibility of the responsible decision-makers, experts and employees.
- ✓ **Interactions with stakeholders** → raising awareness of new knowledge

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CLASSIFICATION AND ASSOCIATED ACTIONS/4

• Several measures can also be implemented to **mitigate remaining and emerging (safety-relevant) uncertainties**:

- ✓ **Implementation of a flexible programme:**
 - to manage e.g. unexpected features or events
 - to keep options open (incl. e.g. retrievability option)
 - optimisation of the repository design - design flexibility and flexibility in design steps
- ✓ **Interactions with stakeholders** to manage e.g. “programme uncertainties”
- ✓ **Implementation of the defence in depth principle** (i.e. one of the fundamental safety principles identified by IAEA):
 - **Protective measures** against disturbing events & processes
 - **Controls & corrective measures** to prevent defects and damages to the disposal
 - **Safety margins**: values of the parameters taken into account in the design are set in order to cover uncertainties related to known unknowns and possibly to unknown unknowns or ignored knowns.
 - **Multi-barrier system** with independent and complementary safety functions and components



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CLASSIFICATION AND ASSOCIATED ACTIONS/5

STEP wise approach

- **Stepwise decision-making process and iterative approach to managing uncertainties as the programme processes through each phase of disposal facility development**
 - ✓ Results from a safety assessment can be used to understand the parameters to which performance measures are most sensitive → provide clues to subsequent data collection activities → reduce the associated uncertainty in a meaningful way
- **Stepwise approach in safety assessment:**
 - ✓ Safety case established in several successive steps, prior to the licence for construction, each of them being reviewed in detail by the regulator and its TSO → opportunity to identify unknown or ignored knowns
 - ✓ The regulator will authorize only operations that will be duly demonstrated as safe
- **Process ensuring regular updates and implementation of R&D programme** in the area of RAW/ NSF management in compliance with the schedule for disposal facility development.

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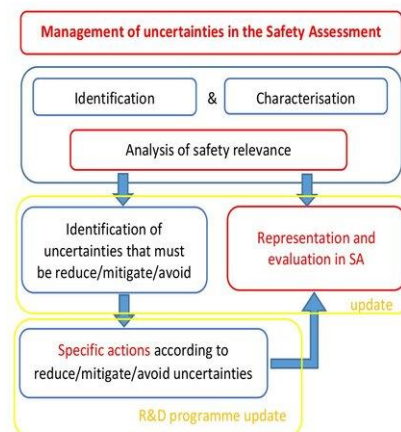


REPRESENTATION IN SAFETY ASSESSMENTS

- **The safety case** as a basic methodology to assure the consistency of all safety-related issues **can be challenged when new evidences, knowledge or techniques are developed.**
- New knowledge is incorporated in the successive versions of the safety assessments
- **The safety assessment can be used to demonstrate the robustness** of the disposal system w.r.t. uncertainties like the occurrence of (unexpected) disturbing events using **dedicated scenarios.**
- **What-if scenarios / analyses** can be used when the cause of the degradation or failure is not known.
- Such analyses contribute to the assessment of the level of **defence in depth** provided by the disposal system.
- **New knowledge should be included/referred in the safety assessment also through the systematic FEP's management**

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CONCLUDING REMARKS ON NEW KNOWLEDGE

The degree of ignorance or acceptance of new knowledge and **willingness to take it into account afterwards in practice** is also related to:

- the degree of responsibility and professional competence and credibility of individual responsible staff or organizations /institutions
- their mutual cooperation regarding knowledge exchanges and, last but not least,
- to the safety culture as such.

The possible impact of new knowledge on safety assessments is expected to decrease over programme phases due to the fact that new knowledge will be taken into account systematically in the periodic safety reviews

Several uncertainty management approaches might be available to avoid, reduce or mitigate some of the “new knowledge” uncertainties. The choice of the approach(es) depends on the type of uncertainty and the programme phase.

For discussion:

To what extent can the potential impact of new knowledge on safety analysis be reduced through its systematic considering within the regular (decennial) safety reviews?

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CONCLUSION/1

- **The uncertainty management strategy should ensure that, at the end of the decision-making process, no remaining uncertainty can potentially jeopardize repository safety.**
- **All methods and technologies selected and used in a RWM programme (disposal programme) have to be defensible with respect to good practice and quality requirements.**
- **The safety case shall specify how uncertainties are identified, how they are characterized and what is the approach for their management !**
- **The choice of an appropriate set of actions may depend on the nature of the uncertainty, i.e. ignored/unknown knowns (available information which is not (properly) considered or known) vs. known unknowns (e.g. the percentage of defects that cannot be detected).**

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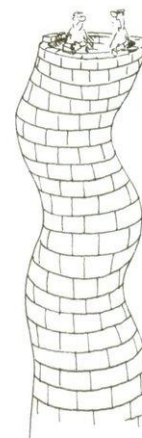




CONCLUSION/2

- How to measure the "immediate benefits" of a given solution when facing intergenerational equity?
- Safety case needs to demonstrate not only "a good system" but also "good processes", namely a "good safety analysis", i.e. it needs to prove that these uncertainties have been dealt with adequately (e.g. QA system, clear roles, etc.).
- Building and/or improving credibility of the responsible organizations and institutions involved in the management of radioactive waste is essential for building public trust in the whole programme implementation, including decision-making process.

Trust is difficult to gain but easy to be lost!



Look, either you believe it or leave it...

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LIST OF RECOMMENDED PUBLICATION

REFERENCES

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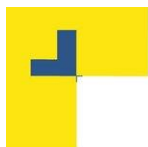
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Thank you for your attention!

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Appendix M. Short career summary of lecturers

This appendix provides a short career summary for each of the lecturers who prepared and gave the lectures of the course.

Valéry Detilleux, Bel V

Valéry DETILLEUX holds a Ph.D. in physical chemistry from the Université catholique de Louvain (Belgium). He joined Bel V in 2011 and works currently at Bel V as a branch manager and a project manager. He is notably involved in the review of safety assessments for radioactive waste treatment, conditioning and disposal facilities. He is responsible for the development and implementation of a R&D programme for supporting the regulatory review of the safety case for a geological disposal facility. He is actively involved in the EURAD Joint Programme, notably as Chairman of the Bureau of the General Assembly. Moreover, he is the SITEX.Network President since January 2022 and participated in several international working groups and projects such as IAEA-GEOSAF II, IAEA-PRISM, EC-FP7-SITEX and EC-H2020-SITEX-II.

Tim Hicks, GSL

Since 1994: Consultant (now Technical Director) at Galson Sciences Ltd, UK (safety of radioactive waste management and disposal)

1988-1994: Researcher at CSM Associates, Cornwall, UK (hot dry rock geothermal energy)

1986-1988: Post-doc University of East Anglia, UK (semi-conductor crystal growth)

1988: PhD at University of Bristol, UK (computational fluid dynamics)

1982: BSc at University of Aston in Birmingham, UK (mathematics)

Work topics:

- Environmental safety case methodology
- Disposal concepts
- Safety assessment
- Criticality safety of fissile wastes

Vincenz Brendler, HZDR

Since 2017: Professorship Radiochemistry at University of Applied Sciences Dresden

Since 2004: Department head at HZDR (RN migration, surface processes, thermodynamics of actinides)

Since 1993: Researcher at Helmholtz-Zentrum Dresden-Rossendorf (HZDR)

1992-1993: PostDoc at NTH Trondheim (Norway)

1993: PhD at Bergakademie Freiberg (Physical chemistry of brines)

1989: Diploma (Chemistry, Bergakademie Freiberg)

Work topics:

- Thermodynamics of aqueous solutions
- Geochemistry
- Molecular understanding of radionuclide migration
- Sensitivity and uncertainty analysis

Klaus-Jürgen Röhlig, TUC

Since 2007: University professor for Disposal Systems at TU Clausthal
2010-2015: Chair of the Integration Group for the Safety Case (IGSC) at OECD/NEA
1991-2007: Gesellschaft für Anlagen- und Reaktorsicherheit (GRS) mbH Köln
(scientific officer, project manager, deputy head of department)
1989-1991: Institut für Energetik Leipzig (scientific officer)
1989: PhD at Bergakademie Freiberg (mathematical analysis)
1985-1989: Bergakademie Freiberg (research fellow)
1985: Diploma (mathematics, Bergakademie Freiberg)

Work topics:

- Safety assessment / safety case methodology
- Analytical assessment of the long-term performance of geotechnical barriers
- Probabilistic methods and sensitivity analysis
- Sociotechnical research: Interface of technical and non-technical aspects in nuclear waste management

Dirk Alexander Becker, GRS

From 1989 to 1993 scientific employee of the Institute for Mathematical Physics of the Technical University of Braunschweig. From 1993 to 1995 employee of the Safety Analyses Group at GSF-Forschungszentrum für Umwelt und Gesundheit GmbH. Since 1995 scientific employee of the Repository Safety Department of the Disposal Division at Gesellschaft für Anlagen- und Reaktorsicherheit (GRS) mbH. Head of group “Statistic Methods”.

Attila Baksay, TSENERCON

Since 2017: Consultant in TS Enercon Ltd.
2011-2017: Physicist at Public Limited Company for Radioactive Waste Management
2008-2011: Physicist at Golder Associates Hungary Ltd.
2003-2007: Physicist at TS Enercon Ltd.
2003: Engineer-physicist diploma at Budapest University of Technology and Economics

Work topics:

- Radioactive waste management techniques
- Safety assessment of radioactive waste management facilities
 - Derivation of radioactive waste inventories (derivation of stochastic inventory)
 - Dose assessments for operational phase of waste management facilities
 - Dose / risk assessment of post closure phase for disposal facilities (stochastic modelling, sensitivity analysis)
 - HAZOP assessments
- Calculations for nuclear safety (burnup, criticality)
- Software development for safety calculations

Peter Schillebeeckx, JRC

Since 2001: Scientific officer at European Commission, Joint Research Centre, Geel, Belgium
1989-2001: Scientific officer at European Commission, Joint Research Centre, Ispra, Belgium
1988-1989: Scientific officer at Institut Laue-Langevin, Grenoble, France
1987-1988: Visiting scientist at the European Commission, CBNM, Geel, Belgium
1984-1987: PhD student, RU Gent and SCK CEN Moll, Belgium

Work topics:

- Neutron detection techniques
- Neutron cross section measurements
- Neutron resonance analysis
- Production of evaluated nuclear data including covariance data
- Non-destructive assay of nuclear material (neutron and gamma-ray detection and calorimetry)

Chris de Bock, ONDRAF/NIRAS

Chris is involved in the management of the acceptance criteria for radioactive waste at ONDRAF/NIRAS, the Belgian Agency for management of radioactive waste and enriched fissile material. He is currently the coordinator of the expertise group charged with the development of the waste acceptance criteria. He is also a task leader in the strategic work package ROUTES of EURAD, a EU joint R&D programme on radioactive waste management. He is leader of Task 4, devoted to waste acceptance criteria. Chris De Bock was previously the project officer at ONDRAF/NIRAS in charge of the cost evaluation of both the geological and surface disposal projects. The cost evaluations provide a key input to the determination of the waste tariffs and budget provisions. Also previously at ONDRAF/NIRAS, he was chairman of the working group on the re-design of the Belgian geological repository and the preparation of its Safety Case. He is co-author of the Safety Analysis Report of the surface repository. Chris participated as Module leader in the 5-year R&D project ESDRED (of the EURATOM 6th Framework Programme), targeted on developing new geological repository technologies. Prior to joining ONDRAF/NIRAS, from 1989 to 2002 Chris De Bock was with Westinghouse Electric Company, where he was active in the design and safety evaluation of nuclear power plants. He managed several smaller and medium-scale study projects for the improvement of safety and/or efficiency in the fields of pressure vessel mechanics, thermo-hydraulics, reactor physics, control instrumentation and also for the development of plant-specific emergency response procedures. Chris De Bock holds a degree in civil engineering (electro-mechanics) from the university of Leuven (Belgium).

Jeroen Mertens, Bel V

Since 2019: Nuclear engineer at Bel V
2007-2018: Inspector-Expert Radioactive waste at FANC
2005-2007: Nuclear safety analyst AVN
2001-2005: Researcher at ONDRAF/NIRAS
2000-2001: Researcher at KULeuven

Work topics:

- Waste acceptance criteria
- 'Disposability' of radioactive waste
- Surface disposal, specifically w.r.t:
 - Licensing,
 - Safety assessment,
 - Design,
 - Uncertainty management.
- Geological disposal

Julien Dewoghélaëre, NTW

Mr. Julien Dewoghélaëre has a graduate degree in Politics Sciences (Sciences Po Bordeaux, 2003). He has been working in MUTADIS from 2011 to July 2022 on research projects and network activities on the governance of hazardous activities, particularly on nuclear related matters. He was involved in the SITEX project (2012-2013) and SITEX-II project (2015-2017) and in the development of tools enabling pluralistic dialogue between various categories of actors on complex topics (including Radioactive Waste Management) entitled Pathway Evaluation Process (PEP). He is currently working

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for NTW and is member of the team in charge of coordinating the Interaction with Civil Society (ICS) activities in EURAD. In UMAN, he is Task leader of TASK 5.

Jeroen Bartol, COVRA

Jeroen Bartol started about five years ago at COVRA, the Dutch waste management organisation, as a research coordinator on disposal in rock salt. He is responsible for the safety case, safety assessment and the cost estimate. He did his bachelor, master and PhD at Utrecht University in geophysics with a focus on numerical modelling.

Jitka Miksova, SURO

Ms. Jitka Mikšová has received her master degree in applied geophysics at Charles University in Prague in 1984. After graduation, she was engaged in the field investigation aimed at water and mineral resources as a prospector, then as a head of the Data Processing Department in a private company. She has been participating in the national RWM programme since 90s when she joined the Czech Geological Survey. Until 2004, as a member of the expert group, she was involved in DGR siting, data and information management, GIS implementation, radon programme establishment, until 2004. Then, more than 10 years she was working for SURAO (Czech WMO) in the Geological Repository Development Department, as a technical development project manager responsible, in particular, for far field research, natural analogues, siting issues, she was involved in project management and strategic documents development. Between 2014 and 2019 she was working in Nuclear Research Centre Rez, in Nuclear Fuel Cycle Department, as a research scientist-expert leading TSO group focusing on RWM. She joined SURO in 2017 (part time), since 2019 she is a head of the Division providing services to the SUJB (Czech regulatory authority) in the field of RWM, she is a member of the SITEX.Network MB, currently participating in EURAD, PREDIS and HARPERS projects and in national research projects on RWM.

Appendix N. Test

This test was prepared by the lecturers and performed by the participants at the end of the course. It aimed at verifying if the audience had broadly understood the lectures and at receiving feedback about the lectures. This is was not a “qualification test”. The correct answers are indicated by a cross in the last column below (this information was provided to the course participants after the test).

A1: Introduction to the safety case, uncertainty management strategy and main types of uncertainties		
<i>Valéry Detilleux</i>		
Q A1-2.1: Uncertainties...		
a	...should not be discussed in the safety case, they have to be discussed in a separate document (not evaluated by the regulatory body).	
b	...are central in the safety assessment and the safety case. The safety case has to present how uncertainties are managed.	X
c	...management in the safety case is a good basis to have a dialogue between the interested partners and develop trust between them.	X
Q A1-2.1: The UMAN work package of EURAD...		
a	...contributes to build a mutual understanding about uncertainty management between the main actors involved in a radioactive waste management programme.	X
b	...identified that it is absolutely impossible to develop a mutual understanding about uncertainty management.	
c	...showed that uncertainties about waste inventory do not have to be managed, they are absolutely not safety significant.	
Q A1-2.3: All the uncertainties that are identified in a radioactive waste management programme can be reduced.		
a	Yes	
b	No, only epistemic uncertainties can (in principle) be reduced.	X
c	No, only uncertainties related to technical aspects can be reduced (and not those associated to human aspects).	

A3: Uncertainty management strategy and approaches		
<i>Tim Hicks</i>		
Q A3.1: A safety case produced at each decision-making stage in a disposal programme, should...		
A	...show that there are no remaining uncertainties that could be significant to safety	

B	...include a plan for how and when any remaining uncertainties that could be significant to safety will be addressed	X
C	...argue that any remaining uncertainties that could be significant to safety will be addressed by the next decision-making stage	
Q A3.2: All uncertainties...		
a	...should be reduced, mitigated or avoided with the objective of ensuring that the detrimental impacts of disposal are as low as is reasonably achievable	X
b	...should be reduced, mitigated or avoided	
c	...should be reduced, mitigated or avoided until they are of no significance to safety	
Q A3.3: Events and processes of irreducible uncertainty regarding their occurrence....		
a	...do not need to be considered in a safety assessment	
b	...can be characterised in terms of their probability of occurrence for inclusion in probabilistic safety assessments	
c	...can be assessed through deterministic or 'what-if' scenarios to understand their potential impacts on safety	X

A4: Uncertainty identification and quantification		
<i>Vinzenz Brendler</i>		
Q A4.1: What is the difference between aleatoric and epistemic uncertainties?		
a	Whether it is (theoretically) possible to reduce the uncertainty or not	X
b	Whether it concerns model or parameter uncertainty	
c	Whether it can be used for sensitivity analyses or not	
Q A4.2: What is the difference between risks and uncertainty?		
a	Risk is a psychological category, uncertainty a scientific one	
b	Risk is uncertainty combined with consequences	X
c	Risk is scenario-based, uncertainty is model-based	
Q A4.3: How can you manage uncertainties that turned out to be not neglectable?		
a	Do more investigations to check whether they can be reduced	X
b	Change system components to get rid of the respective parameter (and thus its uncertainty)	X

c	Search for good reasons why they are still acceptable for the public	

A5: Approaches to uncertainty and sensitivity analysis – Overview		
<i>Klaus-Jürgen Röhlig</i>		
Q A5.1: Sensitivity analysis: Local methods can be based on ...		
a	... calculating the output variance.	
b	... estimating output derivatives.	X
c	... estimating correlation ratios.	
Q A5.2: Sensitivity analysis: Screening methods ...		
a	... are supposed to detect non-monotonic input-output relationships.	
b	... often require specific space-filling sampling schemes and high sample sizes.	
c	... are supposed to identify uninfluential input parameters with relative few model evaluations.	X
Q A5.3: Sensitivity analysis: Variance-based methods ...		
a	... are unable to detect non-monotonic input-output relationships.	
b	... might be able to detect input correlations.	
c	... might be able to detect input interactions.	X

A6: Approaches to uncertainty and sensitivity analysis – Practical application		
<i>Dirk-Alexander Becker</i>		
Q A6.1: What does sensitivity analysis not serve for?		
a	Improving model understanding	
b	Validating the model	X
c	Identifying influential model parameters	
Q A6.2: For polynomial metamodels, how should one choose the maximum polynomial orders?		
a	Not lower than 3	
b	The higher the better	
c	Carefully, depending on the model	X

Q A6.3: What does it mean if the rank-based sensitivity analysis yields, for a specific parameter, a lower (absolute) sensitivity than the direct evaluation?		
a	The parameter acts mainly on high model output values	X
b	The parameter acts mainly on low model output values	
c	The parameter is very uncertain	

B1: Characterization and significance of uncertainties related to the waste inventory <i>Attila Baksay</i>		
Q B1.1: What are the sources of the uncertainties related to radioactive waste inventory?		
a	Waste stream identification, Waste production rate, Radiological- Physico - Chemical Characterization, Waste processing (Pre-treatment, Treatment, Conditioning), Long-term physico-chemical interactions in Storage and Disposal	X
b	Only the radiological characterization provides significant uncertainties in the radioactive waste inventory.	
c	Scaling factor methodology used in radioactive waste characterization. Chemical properties of the waste. Long term behavior of the waste packages related to the final disposal.	
Q B1.2: In which areas does the uncertainty about the waste inventory play a role?		
a	The operational and post closure safety of the radioactive disposal sites is the related issue that can be effected by these uncertainties.	
b	Basically every aspects of radioactive waste management; operational and post-closure safety of the facilities, design of these facilities, selection of waste management routes. The uncertainties also have impact on the optimization of the predisposal and disposal of the radioactive waste.	X
c	The operational safety of the radioactive waste management facilities is the main issue, where the accidents related to waste treatment can be prevented.	
Q B1.3: What efforts are to be done in order to minimize the uncertainties related to the uncertainties of radioactive waste inventory?		
a	Development of radiological characterization. Improvement of Quality Assurance and Quality Control of the radiological characterization.	
b	The efforts of organisations involved in radioactive waste management to minimise these uncertainties cannot be foreseen. The scope for speculation is too great.	
c	Development in methodologies and technologies of radiological and non-radiological characterization. Efforts for multiple characterization of waste packages through the predisposal steps. Improvements in statistical methodology related to better process measured data. Improvement of Quality Assurance and Quality Control of the	X

	characterization and in the process of waste processing. Data collection in order to understand storage systems and disposal systems.	
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B2: Uncertainties in the radiological characterization of spent fuel

Peter Schillebeeckx

Q B2.1: What is the best procedure to estimate the decay heat rate (or decay power) of a spent fuel assembly together with the long timescale evolution of the decay power (> 100 year)?

a	Determine the inventory of ⁹⁰ Sr, ¹⁰⁶ Ru, ¹³⁷ Cs, ¹⁴⁴ Ce, ²³⁸ Pu and ²⁴¹ Pu from a representative sample by radiochemical analysis and use this inventory to calculate the decay power as a function of cooling time.	
b	Determine the decay power of the assembly by measurements with a calibrated and validated calorimeter and extrapolate the result for longer timescales.	
c	Use a validated depletion code to estimate the nuclide inventory at the end of irradiation in the reactor and use this inventory to calculate the decay power as a function of cooling time	X

Q B2.2: What is the minimum relative uncertainty of the calculated neutron emission rate due to spontaneous fission of a PWR spent nuclear fuel assembly with a cooling time of 10 years? The calculations are performed with the following input data: IE (initial enrichment) = 4.480 (45) wt% and BU (burnup) = 45.0 (9) MWd/kg.

a	2%	
b	4%	
c	8%	X

Q B2.3: A measurement of a spent nuclear fuel sample with a calorimeter results in a decay power of 230 (9) W. An estimation of this decay power (for the same cooling time) using a reference method results in (300 ± 6) W. What is the estimated systematic error of the result of the calorimeter? Note that the uncertainties are given following the recommendations in the GUM.

a	- 70.0 (95) W	X
b	- 70 (11) W	
c	70.0 (95) W	

B3: Uncertainties associated with waste management routes

Chris De Bock

Q B3.1: What can be considered to be the essence (“backbone”) of radioactive waste management ?

a	The characterization of the radioactive waste	
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b	The definition, for each waste type, of a route of activities from the generation of the waste to its disposal	X
c	The availability of a radioactive waste disposal facility	
Q B3.2: Name two important disadvantages of early conditioning (without an established disposal route)		
a	<ol style="list-style-type: none"> 1. Uncertain qualification of the conditioning process 2. Need to keep the conditioned waste packages in storage for a long period of time 	
b	<ol style="list-style-type: none"> 1. Not being able to benefit from future more efficient conditioning techniques 2. Increased risk of degradation of the conditioned waste packages during storage 	
c	<ol style="list-style-type: none"> 1. Uncertain acceptability of the conditioned waste packages for the disposal facility 2. Early (up-front) investments 	X
Q B3.3: What is an important prerequisite to developing shared solutions ?		
a	All participating countries must be countries with a small radioactive waste inventory.	
b	A common safety culture and a level playing field must exist between participating countries.	X
c	All participating countries must have a certain level of creditworthiness.	

B.5.: Options for the management of uncertainties related to the waste inventory		
<i>Jeroen Mertens</i>		
Q B5.1: Which of the following is a way in which the uncertainties related to chemical composition of the waste could be represented in the Safety Assessment		
a	By performing laboratory experiments that reduce the uncertainties	
b	By using conservative assumptions/parameter settings that envelope the uncertainties	X
c	By listing them as uncertainties that will be dealt with in a next iteration of the Safety Assessment	
Q B5.2: What is a way in which uncertainties related to physical-chemical conditions in the disposal installation could be reduced		
a	By performing monitoring of the above ground storage conditions (t°, humidity)	
b	By representing the uncertainties in altered evolution scenarios	
c	By performing mock-up experiments that simulate the physico-chemical environment of the disposal	X

Q B5.3: What uncertainties related to waste inventory were seen by the Waste management organisations/Technical support organisations/Research Entities as not very important		
a	Uncertainties related to radionuclide activity	
b	Uncertainties related to chemical composition	
c	Uncertainties related to voids in the waste	X

C1: Views of civil society on uncertainty management		
<i>Julien Dewoghelaere</i>		
Q C1.1: Why is it interesting to implement interactions between experts and civil society in the perspective of uncertainty management?		
a	It is a legal requirement	
b	Such interactions will help to develop a mutual understanding of the uncertainties at stake and to disseminate a shared safety culture	X
c	There is no interest	
Q C1.2: Regarding civil society views on human uncertainties, which of the following statement is true (based on the UMAN results)?		
a	Public acceptance should be understood as a way to guarantee the quality of the uncertainty management rather than an uncertainty in itself	X
b	It is possible to postpone decisions endlessly without any alternatives in mind	
c	The apparition of new knowledge undermines the credibility of the safety review	
Q C1.3: UMAN project identified methodologies to implement pluralistic management of uncertainties. Regarding these methodologies, which of the following statement is true?		
a	The involvement of civil society in the management of uncertainty does not require an appropriate legal framework	
b	The Pathway Evaluation Process (PEP game) is the method for the involved actors to choose the best technical solution	
c	Long-term/rolling stewardship, understood as a well-structured governance system could help ensuring intergenerational transmission of information, empowerment of communities and cultural heritage	X

C2: Characterization and significance of uncertainties for different categories of actors		
<i>Jeroen Bartol</i>		
Q C2.1: During the identification of the uncertainties, each uncertainty was assigned an category. Which was not a category?		
a	Societal	

b	Political	
c	Safety	x
Q C2.2: Which of the uncertainties here below were considered in the list of the 10 uncertainties that were elaborated on within more details and were used in the questionnaire?		
a	Continuity of the waste management policy along political changes	x
b	Human intrusion	
c	Public acceptance of the repository at potentially suitable or projected location	x
Q C2.3: What should you always do when you make a decision to include or exclude an uncertainty from the list of uncertainties that need further research?		
a	Nothing.	
b	Record why you made that decision and arguments why it was excluded or included and include this in the final report.	x
c	Make a note of why an uncertainty is included or exclude but do not put this in the final report.	

C3. Options for the management of uncertainties related to human aspects		
<i>Jitka Miksova</i>		
Q C3.1: Is it a public acceptance one of the important factors for a successful siting process? Has a public acceptance a direct impact on a disposal facility safety?		
a	The public acceptance is important and has a direct impact on the facility safety	
b	The public acceptance is important and but has not a direct impact on the facility safety	x
c	The public acceptance is not important and has a direct impact on the facility safety	
Q C3.2: Can uncertainty in the financial and raw resources affect the schedule of the RW programme implementation? Can be this uncertainty in the schedule avoided?		
a	An uncertainty in the financial and raw resources can affect the schedule and can be avoided.	
b	An uncertainty in the financial and raw resources can affect the schedule and can be avoided.	
c	An uncertainty in the financial and raw resources can affect the schedule and cannot be avoided.	x
Q C3.3: Should the safety case specify how uncertainties caused by human factors are identified, how they are characterized and what is the approach for their management? Should be these uncertainties evaluated within Periodic Safety Review?		

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a	The management of uncertainties caused by human factors is not a part of safety case thus there is no need to periodically evaluate them within Periodic Safety Review.	x
b	The management of uncertainties caused by human factors should be an integral part of safety case but there is no need to evaluate them within Periodic Safety Review.	
c	The management of uncertainties caused by human factors should be an integral part of safety case and should be periodically reviewed.	