

#### Spent Fuel Characterization and Evolution Until Disposal

Joaquin Cobos/CIEMAT, on behalf of the WP coordination team Berlin, 3-4 December 2018

IGD-TP EF8 3-4 December 2018

|                  | Name           | Organisation          | Country | Type<br>(WMO,<br>TSO, RE) |
|------------------|----------------|-----------------------|---------|---------------------------|
| Main coordinator | Peter Jansson  | Uppsala<br>University | SE      | RE                        |
| Subcoordinator   | Stefano Caruso | NAGRA                 | СН      | WMO                       |
| Subcoordinator   | Joaquin Cobos  | CIEMAT                | ES      | TSO                       |

## **Content of presentation**

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- 6. WP Tasks' descriptions
- Key challenges and objectives in the first year to communicate near-term focus

## **SFC - Main Objectives**

- Experimental procedures to estimate the isotopic content of SNF & uncertainties.
- Establish <u>characterization techniques</u> for SNF during its evolution from reactor to disposal.
- **Uncertainty quantification** on the evolution of SNF in pre-disposal activities.
- Performance of the SNF during pre-disposal, to build basis for safety concepts/culture.
- Ageing effect & behavior of fuel, cladding, fuel-cladding interaction under normal and accident conditions.
- Contribute to the operational safety concepts for fuel handling at SNF packaging facilities.
- **Education, training** and building **competences** in the subject.

## **WP** Expected impacts

#### Radiactive Waste Management implementation needs

Reduced uncertainty of the decay heat conomical savings DGR.

#### Safety Analysis and Assessment

- Reduced uncertainties on safety.
- Guidance on operational safety of interim storage & fuel packaging facilities.
- Safety assessment on pre-disposal technologies could predict the consequences of design changes.
- Increasing scientific and technical Knowledge in RWM
  - Fuel & cladding: thermomechanical/chemistry degrading mechanisms.
  - Identification gaps and propose beyond the SOTA.
  - Increase knowledge to the European concept/strategy.
  - Models and experimental understanding of the SNF-rod behavior .
- Radioactive waste management routes
  - WP will meet WMO's, regulatory bodys, RE's and TSO's Stategy.
  - Contribute to implement EU directives and comunications.

### **SFC** Participants

#### **Organisations**

- ✓ **CEA**, France (RE)
- ✓ **CHRDI**, Ukraine (WMO)
- ✓ **CIEMAT**, Spain (TSO)
  - IDOM, Spain
  - UPM, Spain
- ✓ **CNRS**, France (RE)
  - Umontpellier, France
- ✓ ENRESA, Spain (WMO)
  - ENUSA, Spain
- ✓ **CPST**, Lithunia (TSO)
- ✓ **FZJ**, Germany (RE)
  - HZDR, Germany
- ✓ **IRSN**, France (TSO)
  - NTW, France
- ✓ JRC, Belgium (RE)
- ✓ **JSI**, Slovenia (TSO)
  - EIMV, Slovenia

#### Organisations

- ✓ **KIT**, Germany (RE)
  - BAM, Germany
  - PEL, Germany
- ✓ LEI, Lithuania (RE)
- ✓ MTA EK, Hungary (RE)
- ✓ **NAGRA,** Switzerland (WMO)
- ✓ **PSI**, Switzerland (RE)
- ✓ **SCK-CEN**, Belgium (RE)
- ✓ **SKB**, Sweden (WMO)
  - UU, Sweden
- ✓ **SSTC NRS**, Ukraine (TSO)
- ✓ **SURAO**, Czech Republic (WMO)
  - CTU, Czech Republic
- ✓ TUS, Bulgaria (RE)
- ✓ VTT, Finland (TSO)

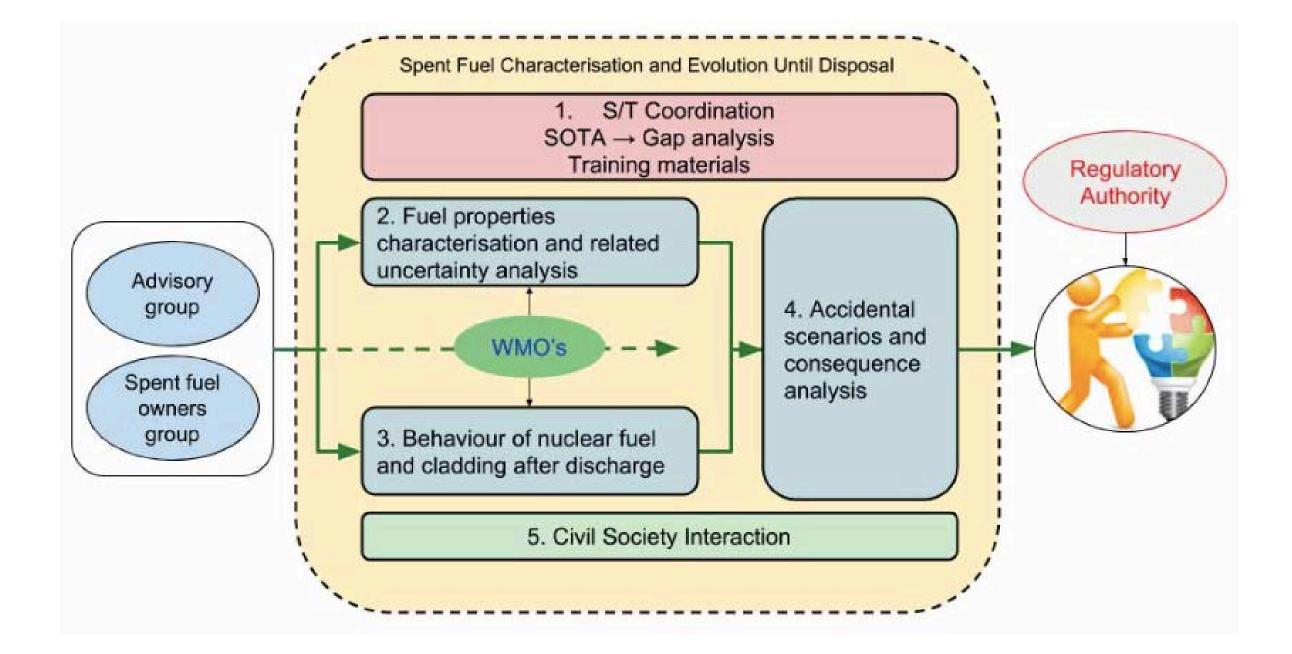
## **Interested external parties**

| Organisation  |  |
|---|--|
| Chalmers University of Technology   |  |
| Czech Radioactive Waste Repository Authority (SÚRAO)  |  |
| Electric Power Research Institute (EPRI)  |  |
| Endesa  |  |
| Gesellschaft für Anlagen- und Reaktorsicherheit (GRS)   |  |
| Italian National Agency for New Technologies, Energy and Sustainable Economic Development<br>(ENEA) |  |
| Kharkov Institute of Physics and Technology   |  |
| Lawrence Livermore National Laboratory  |  |
| National Nuclear Laboratory   |  |
| Oak Ridge National Laboratory   |  |
| OECD NEA Data Bank  |  |
| Pacific Northwest National Laboratory   |  |
| Radioactive Waste Management (RWM)  |  |
| Studsvik (SCIP project)   |  |
| Swedish Academic Initiative on Nuclear Technology Research (SAINT)                                  |  |
| TS Enercon  |  |
| University of Bristol   |  |
| Tokyo Institute of Technology   |  |

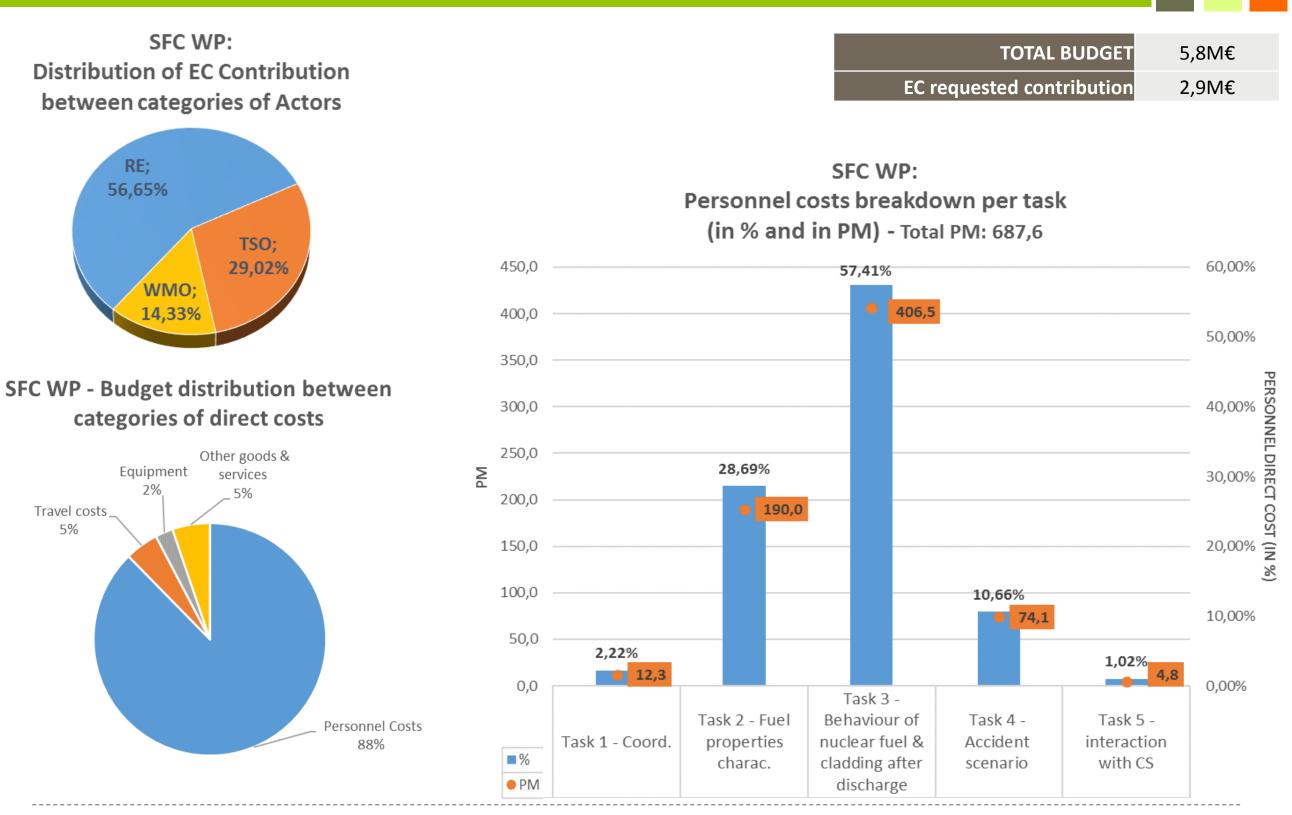
## WP – Task Breakdown and WP Board

- Main coordinator, Peter Jansson, Uppsala University
- Contact info: <u>peter.jansson@physics.uu.se</u>, +46184715841
- Task I S/T coordination, State-of-the-art and training material.
  - ► Task Leader (SKB/UU)
- Task 2 Fuel properties characterisation and related uncertainty analysis Start.
  - Task Leader (JRC Geel/SCK-CEN)
- Task 3 Behaviour of nuclear fuel and cladding after discharge.
  - Task Leader (KIT/BAM)
- Task 4 Accident scenario and consequence analysis
  - Task Leader (CIEMAT/NAGRA)
- Task 5 Civil Society (CS) interaction
  - Task Leader (JSI/EIMV)

#### Structure of the SFC WP



### **WP – Planned resources**



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WP – Task 1 – S/T coordination, State-of-the-art & training material

**Task Leader**: SKB (UU)

#### • Objective:

- Provide efficient management and administration of the work package
- Identify potential problems at an early stage and provide timely and effective solutions.
- Developing/updating SOTA and training materials as an input to KM.
- Subtask I.I S/T coordination
- Subtask I.2 State-of-the-art and Gap analysis
- Subtask I.3 Training materials

# WP – Task 2 - Fuel properties characterisation and related uncertainty analysis

- **Task Leader**: JRC- Geel
- Objective: Produce experimentally verified procedures to determine reliable source terms of spent nuclear fuel (SNF), including realistic uncertainties.
  - Subtask 2.1 Theoretical study of SNF source terms: Timescale 48 months. Subtask Leader PSI
  - Subtask 2.2 Develop, improve and demonstrate NDA methods/systems for SNF characterisation. Timescale 48 months.
     Subtask Leader SCK.CEN
  - Subtask 2.3 Determine the inventory of activation and fission products in cladding material. Timescale 48 months. Subtask Leader VTT.
  - Subtask 2.4 Define and verify procedures to determine the source terms of SNF assemblies with realistic confidence limits. Timescale 35 months. Subtask Leader KIT.

# WP – Task 3 - Behaviour of nuclear fuel and cladding after discharge

#### **Task Leade**r: KIT (BAM)

- Objective: Understand and describe numerically the behavior of SNF, irradiated cladding, fuel/cladding chemical interaction (FCCI) and ageing effect under conditions of extended interim storage, transportation and emplacement in a final disposal system.
  - Subtask 3.1 Thermo-mechanical-chemical properties of the SNF rods and cladding Timescale 48months. Subtask Leader CIEMAT (UPM).
  - Subtask 3.2 Behaviour of SNF pellets under interim storage conditions. Timescale 48 months. Subtask Leader CIEMAT.
  - Subtask 3.3 Pellet-cladding interaction under conditions of extended storage, transport and handling of SNF rods. Timescale 48months.
     Subtask Leader CIEMAT

- **Task Leader**: CIEMAT (NAGRA)
- Objective: Study SNF behaviour under accident conditions which may lead to a potential loss of confinement during storage, transport and pre-disposal activities.
  - Subtask 4.1 Accident scenario for fuel under dry interim storage conditions Timescale 48 months. Subtask Leader NAGRA.
  - Subtask 4.2 Consequence analysis of accident scenarios
    35 Months. Subtask Leader ChRDI.

- **Task Leader**: JSI (EIMV).
- **Objective**: To follow by the CS the work conducted in the WP by using a CS interaction framework developed and established for EURAD, facilitating the translation of results and other output of the WP to the civil society. Finally the task 5 will enhance the possibilities of CS participation in future situations where there are consultation processes as a part of safety case review.
  - Subtask 5.1– Scoping of SFC tasks 2-4, initial input from the CS experts and development of Interaction with Civil Society (ICS) action plan. Timescale 12 Months.Task Leader [JSI (EIMV)].
  - Subtask 5.2– Implementing ICS action plan. Timescale 24 Months. Task Leader [JSI (EIMV)].
  - Subtask 5.3– Synthesis work and dissemination. Timescale 24 Months. Task Leader [JSI (EIMV)].

### Key challenges & objectives for Year 1

- Task I:
  - S/T coordination startup.
  - Focus on gap analysis as well as literature research, aiming to collect and summarise SOTA research activities
    - Deliverable: State-of-the-art report M6
- Task 2
  - Definition of the different necessary quantities to perform the code-tocode.
  - A selection of a number of representative assemblies will be made.
  - Currently used NDA techniques for SNF to be described.
  - Preparation of irradiated sub-samples of cladded UO<sub>2</sub> and MOX pellets to be measured.

### Key challenges & objectives for Year 1

- Task 3:
  - Preparation of irradiated and unirradiated samples of spent nuclear fuel rod segments and cladding.
  - Start of experiments to identify the behaviour of irradiated fuels and simulated spent nuclear fuel pellets.
- Task 4:
  - Identification and first analysis of credible accident scenarios related to transport/storage of SNF.
  - Activities devoted to link this task to the Task 2 and 3 are also foreseen.
    - Deliverable: state-of-emergency radioactive waste M3
- Task 5:
  - Scoping of the objectives and actions in SFC tasks 2-4 in order to identify issues that are deemed of more specific interest in the perspective of developing interactions between civil society and EURAD partners along the course of the WP.
    - Deliverable: ICS action plan MII