

The ENEA logo features the word "ENEA" in a bold, white, sans-serif font. To the left of the text is a stylized graphic of a sun or starburst with a bright yellow center and a red and orange glow, set against a dark blue background with a grid pattern.

AGENZIA NAZIONALE
PER LE NUOVE TECNOLOGIE, L'ENERGIA
E LO SVILUPPO ECONOMICO SOSTENIBILE

IGD-TP 4th Exchange Forum

WG3: New Waste Type, in collaboration with SNETP

Long-term Behavior of Waste Forms from Gen IV Reactors towards Geological Disposal

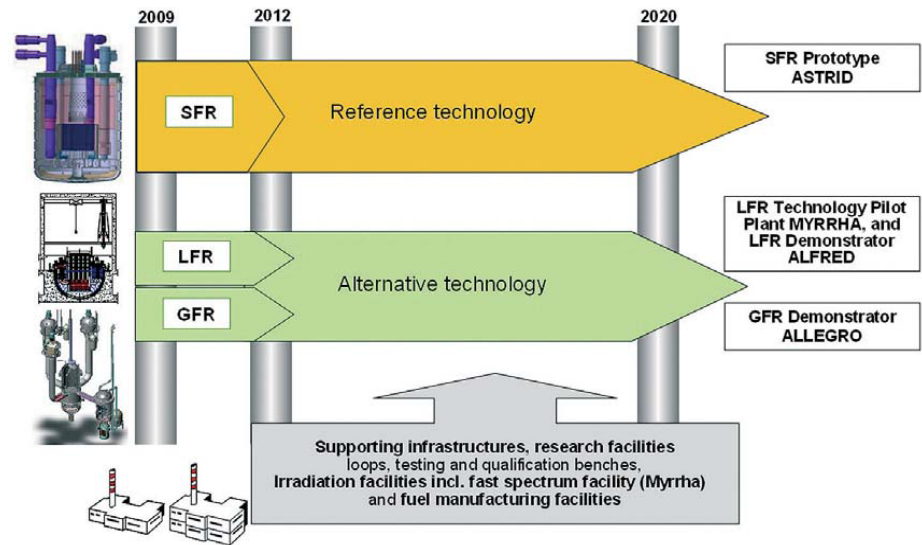
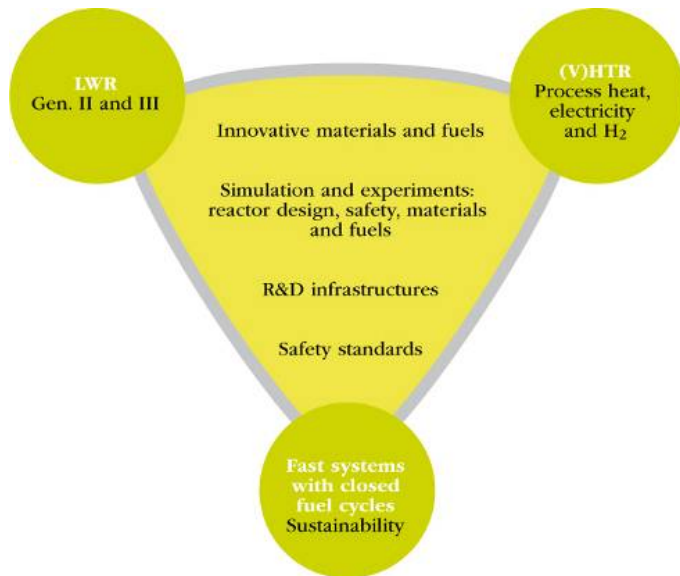
G. De Angelis, A. Dodaro, M. Sepielli – ENEA, UTFISST

Prague, October 29-30, 2013

SNETP pillars and IV Generation Technologies



NUGENIA



ESNII



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Coordinating energy research
for a low Carbon Europe

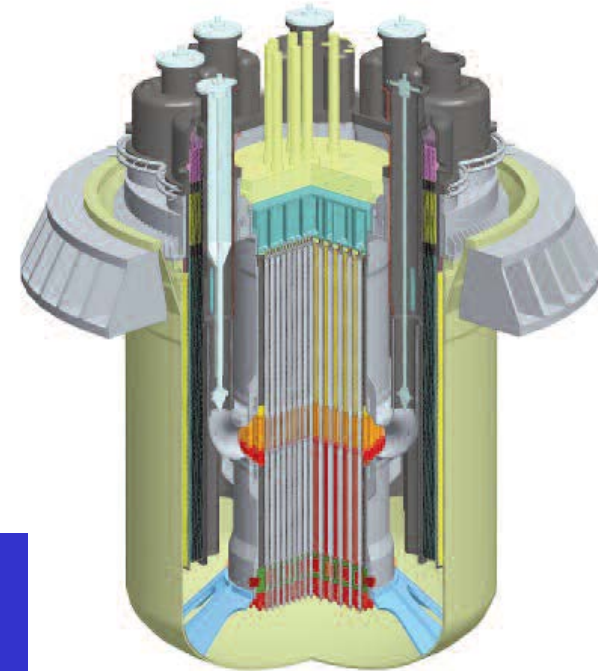
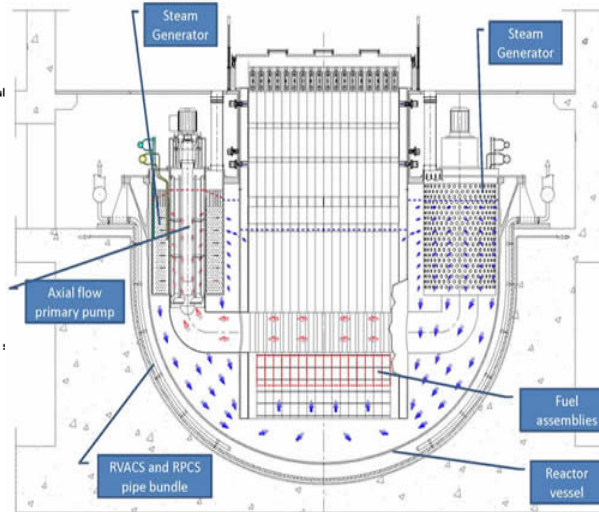
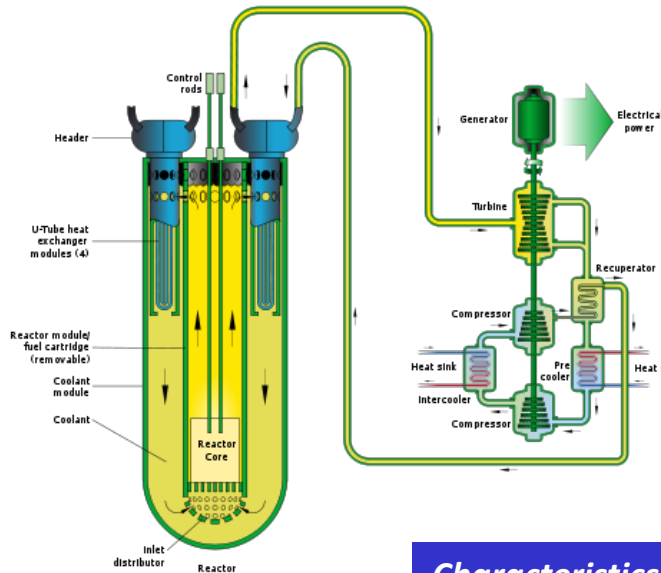
LFR – Lead Fast Reactor Demonstrator - ALFRED



Agenzia nazionale per le nuove tecnologie, l'energia
e lo sviluppo economico sostenibile



AnsaldoNucleare



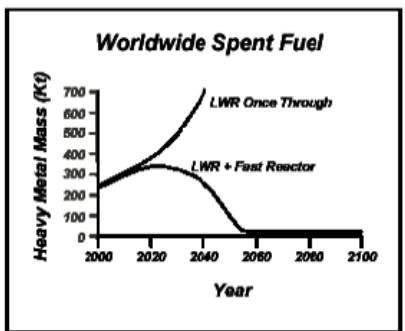
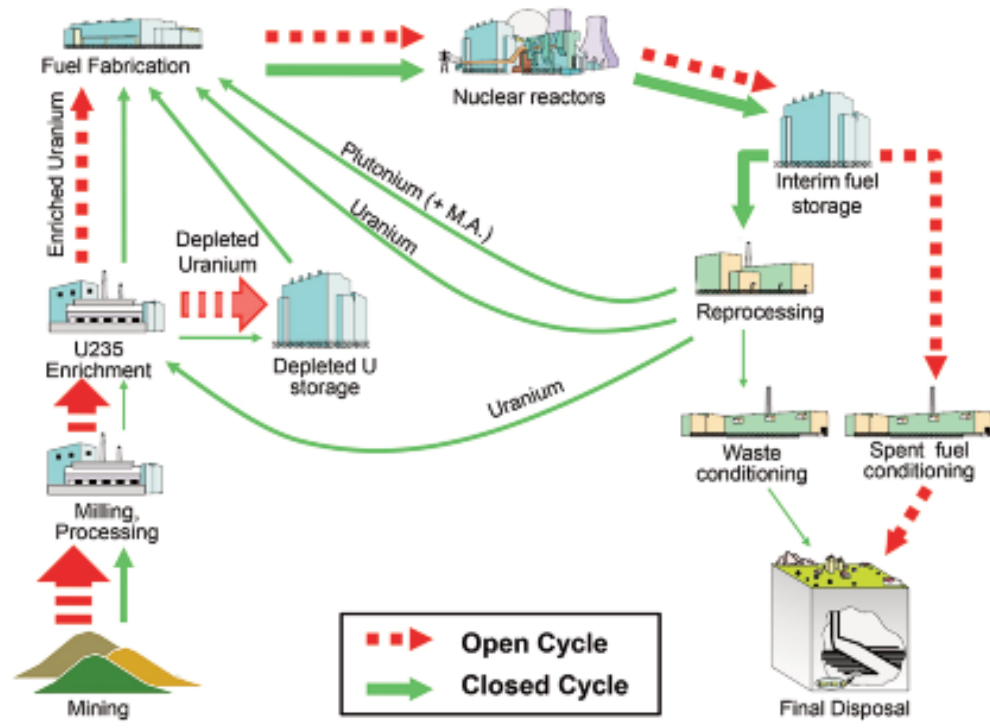
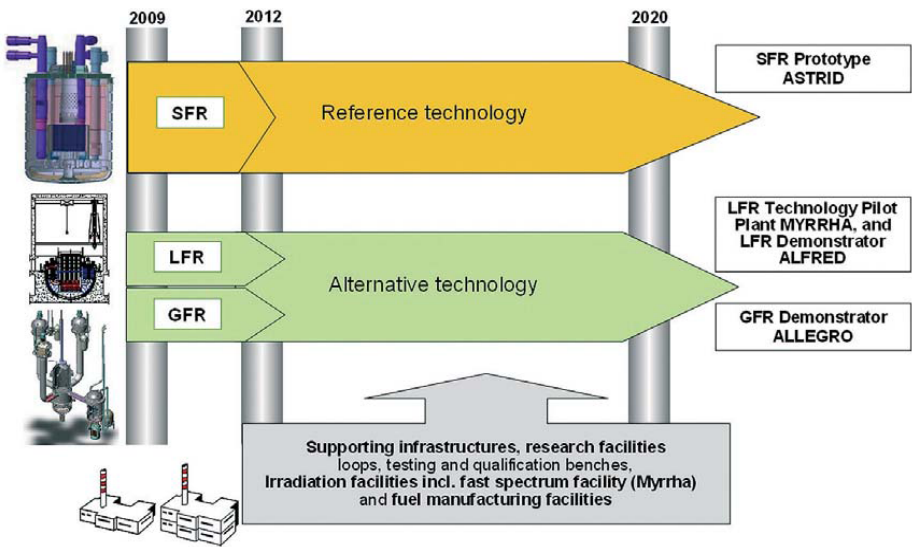
Characteristics

- Pb or Pb/Bi coolant
- 550 ° C to 800 ° C outlet temperature
- Fast Spectrum
- Multi-TRU recycle
- 50–1200 MWe
- 15–30 year core life

Gen IV reactors spent fuel reprocessing

ESNII prototypes

Fuel reprocessing



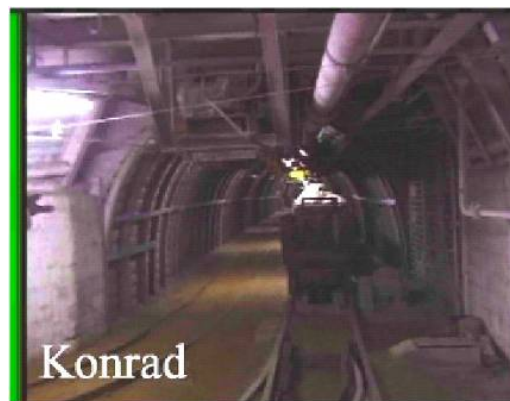
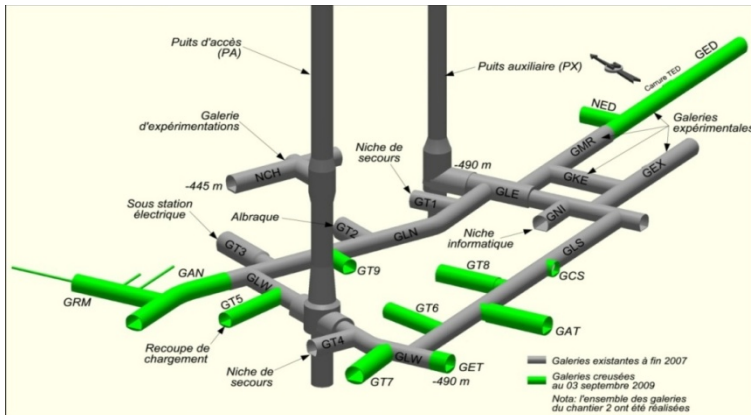
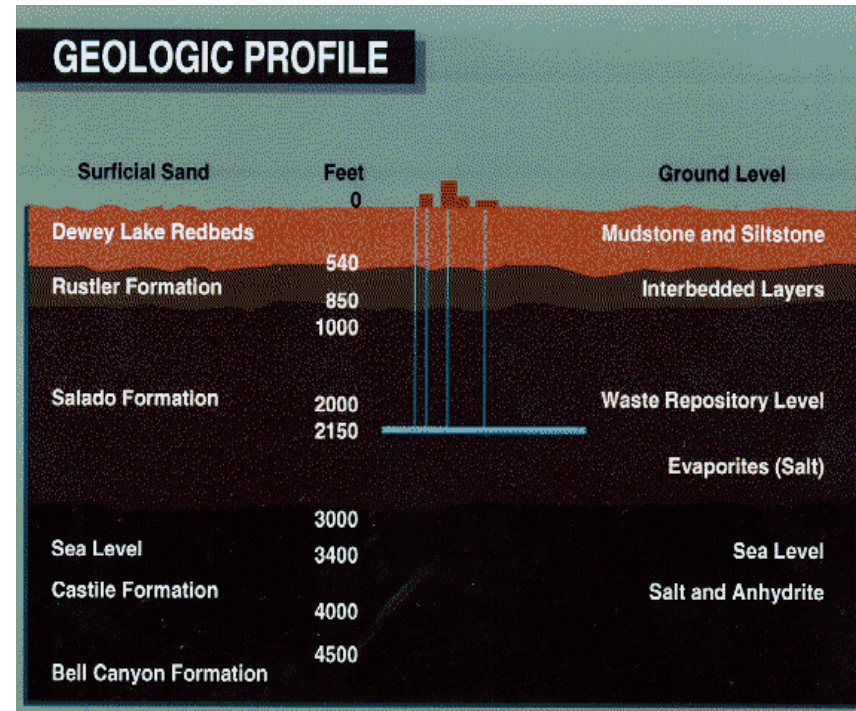
Fuel Cycle Innovations



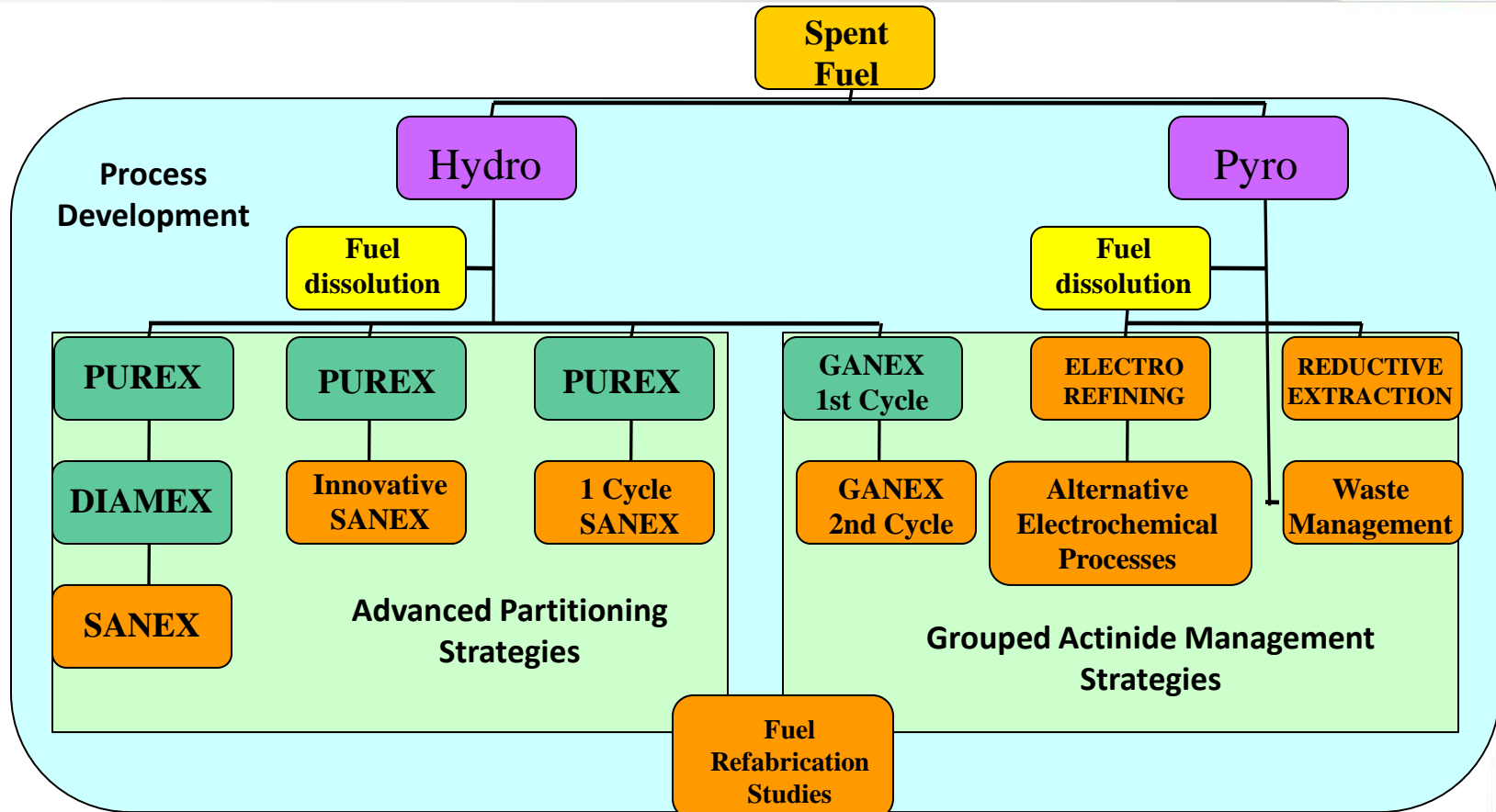
- *Advanced aqueous or pyroprocess recycle technology with intrinsic nonproliferation features*
 - *Produces co-mixed Pu+MA feedstock (no separated Pu)*
 - *Incomplete fission product separation*
- *Fuel Cycle Support Facilities Options*
 - *Co-located*
 - *Regional*
- *Multiple Fuel Options*
 - *Fuel types TRU/U/Oxide, TRU/U/Nitride, TRU/U/Alloy, TRU Alloy, MA Alloy*
 - *Remote refabrication technology depends on fuel type*
 - *Simplified pelletization (oxide, nitride)*
 - *Vibrocompaction (oxide, nitride)*
 - *Injection casting (metal)*

New waste form requirements from Gen IV reactors reprocessing to geological disposal

- Burn most of TRU / Minor Actinides
- Reducing FP as much as possible
- Non proliferation of Pu
- Stability of chemical and physical form
- Easy conditioning and storage
- Safe and economic geological disposal
- Average depth around m.500 and not m. 4000



Treatment of spent fuel



Process Integration

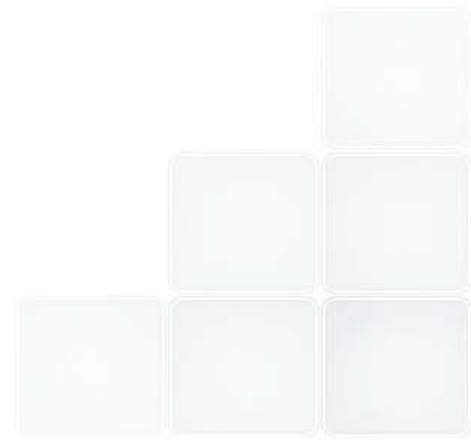


Other European Projects

Assessment of reprocessing flowsheets for different fuel cycle scenarios with a view to their future demonstration at the pilot level

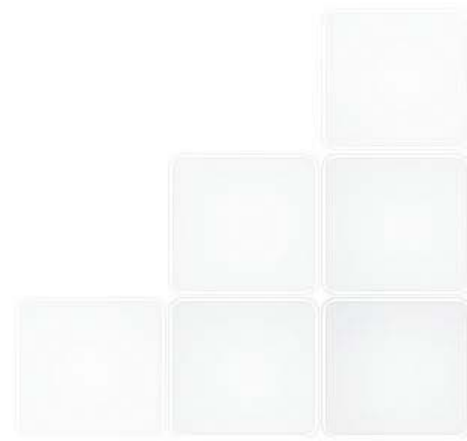
The main reprocessing methods presently used:

- **Co-extraction**
- **New extraction**
- **Uranium extraction (UREX)**
- **Electrochemical processing**



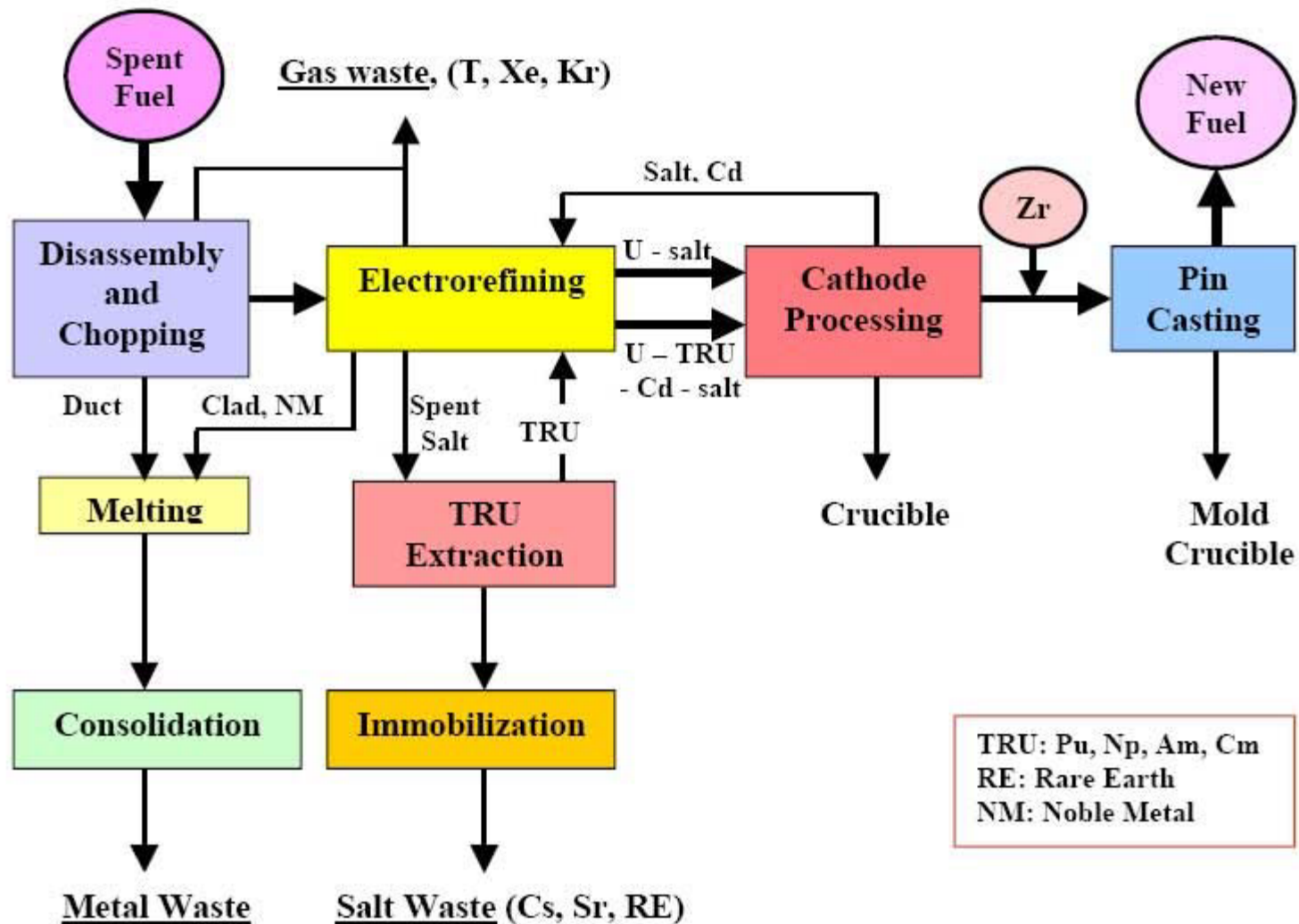
WG3: New Waste Type

Electrochemical processes



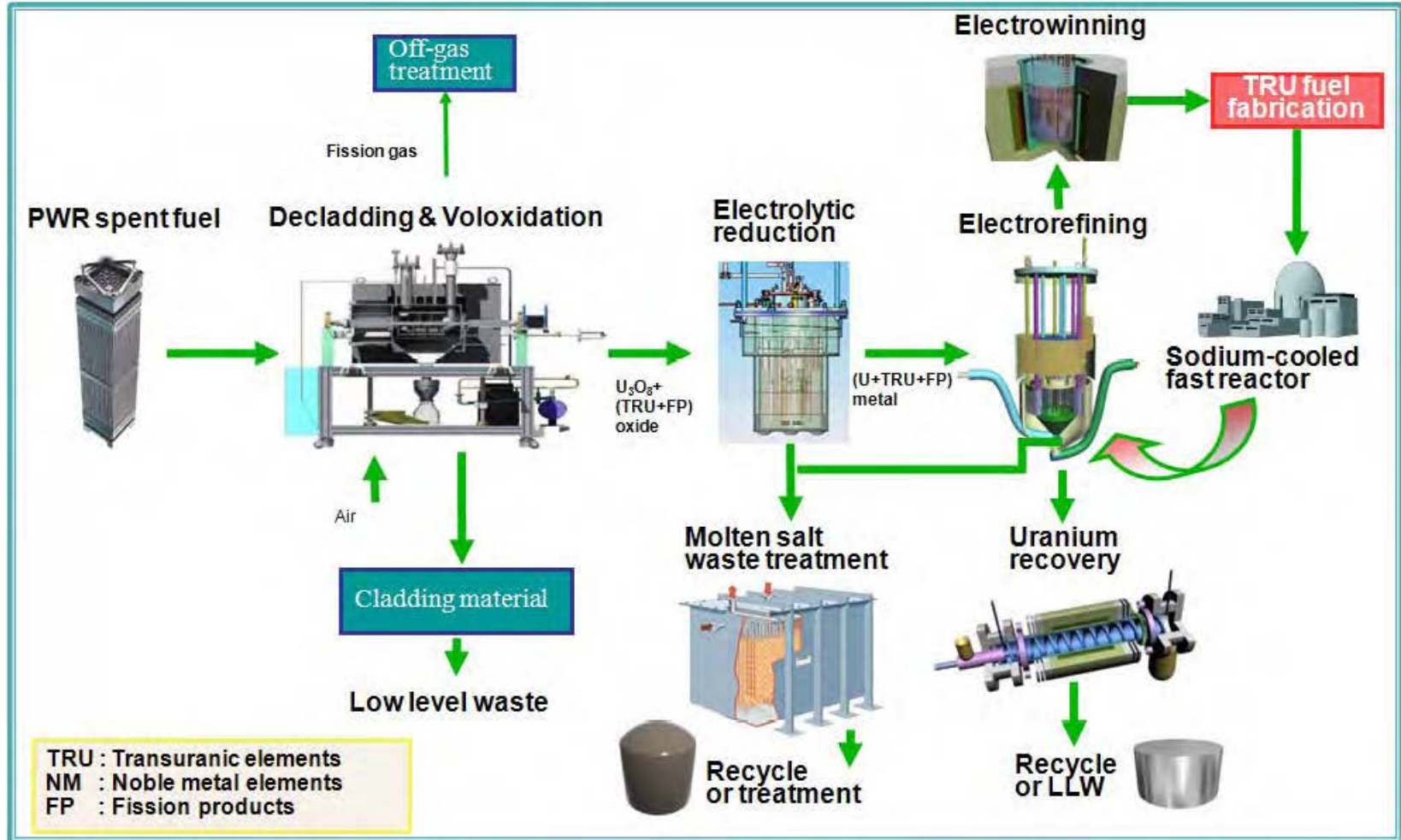
WG3: New Waste Type

Block diagram of the pyrochemical process for metal fuel



WG3: New Waste Type

Schematic illustration of the pyrochemical process for oxide fuel

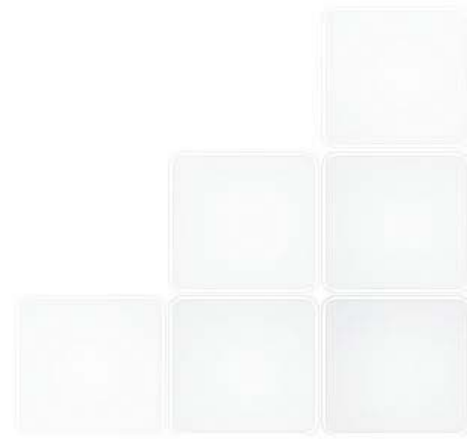


**Conditioning of chloride salt wastes from pyroprocesses
with different matrices:**

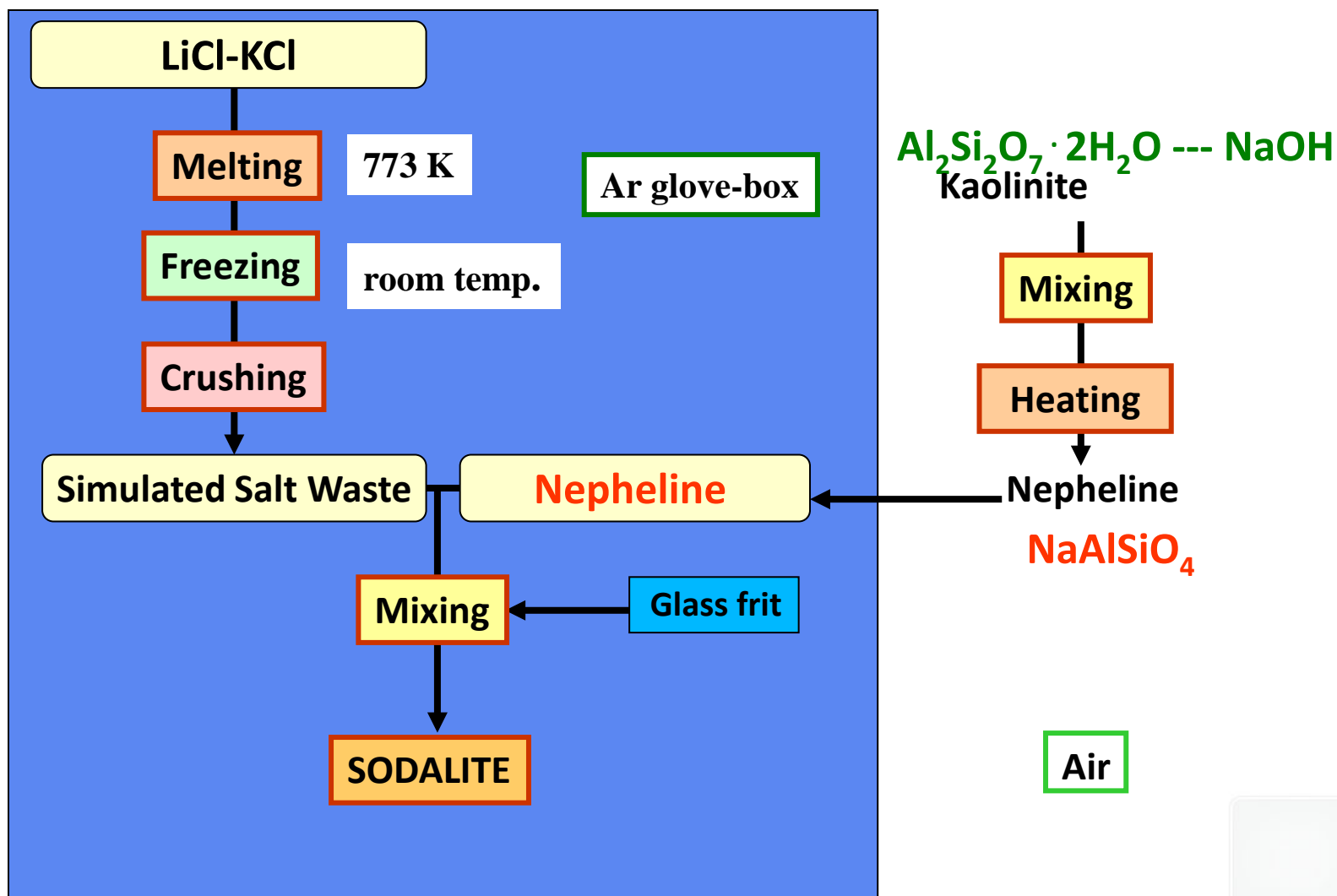
SODALITE

SAP

MURATAITE-PYROCHLORE

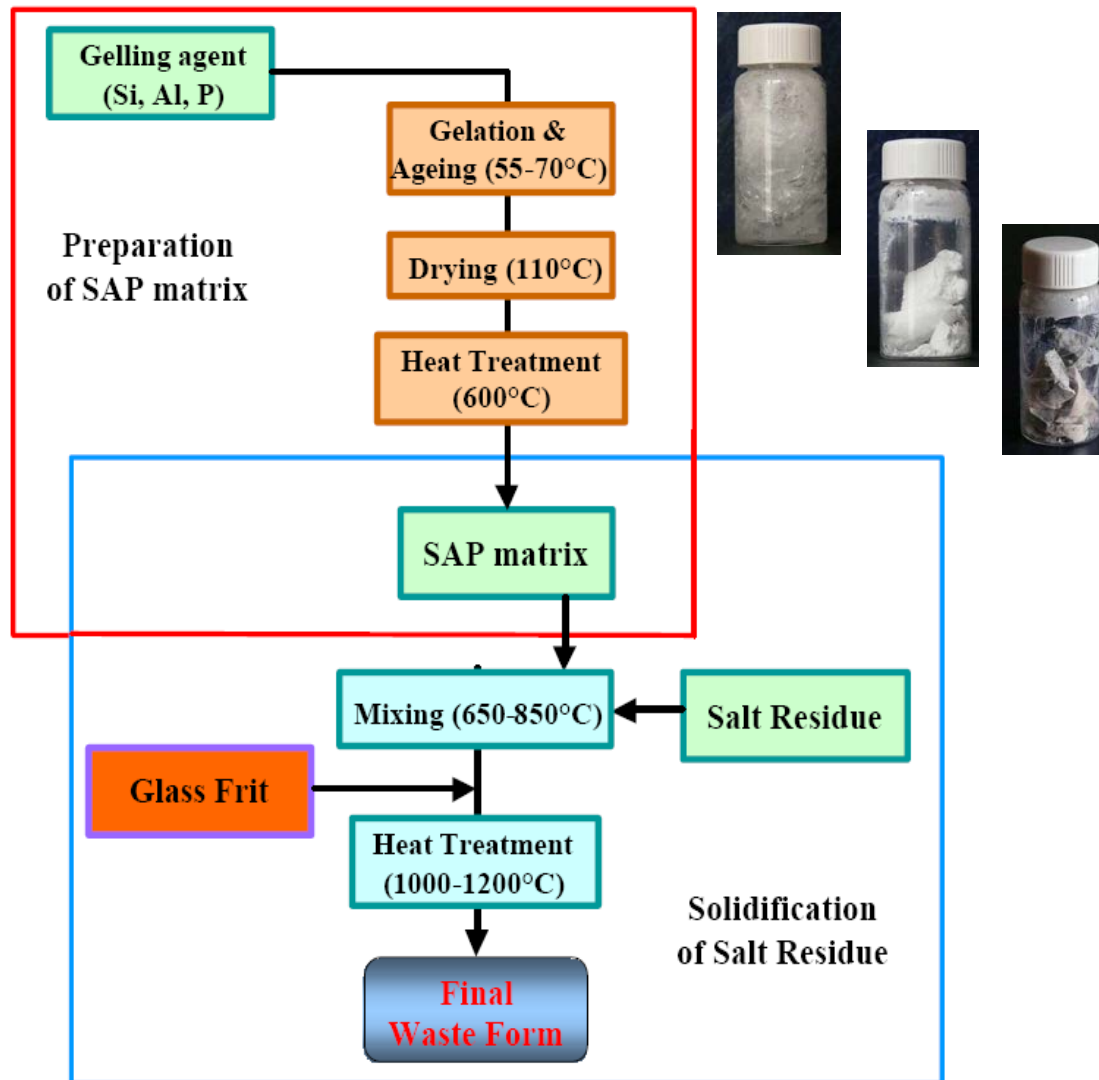


SODALITE matrix



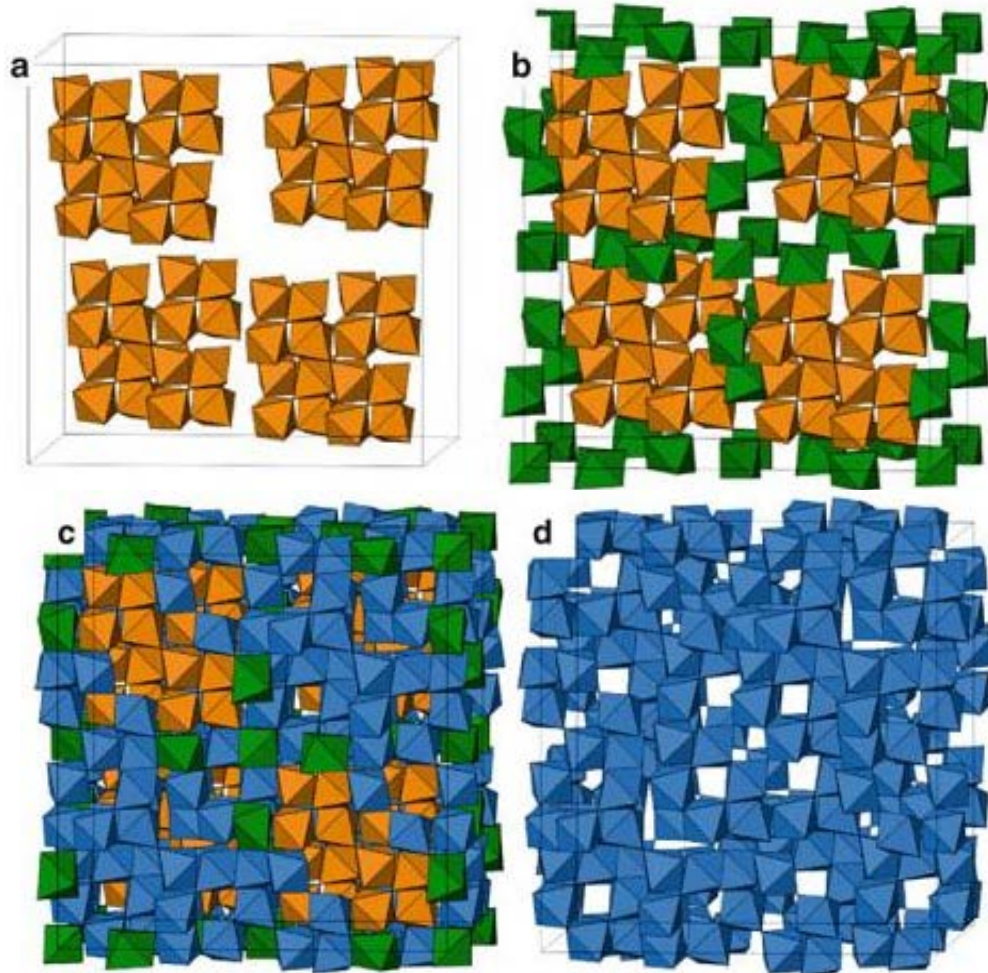
Outline of sodalite synthesis from kaolinite through nepheline

SAP matrix



Outline of SAP synthesis by a conventional sol-gel process

MURATAITE matrix



Octahedral framework in the structure of Mu-5: arrangement of pyrochlore clusters formed by corner sharing of $Ti_{10}O_6$ and $Ti_{40}O_6$ octahedra (a); linkage of pyrochlore clusters by $Ti_{30}O_6$ octahedra (b); whole framework as combination of linked pyrochlore clusters and murataite-like framework formed by $Ti_{20}O_6$ and $Ti_{50}O_6$ octahedra ©; murataite-like framework (d)

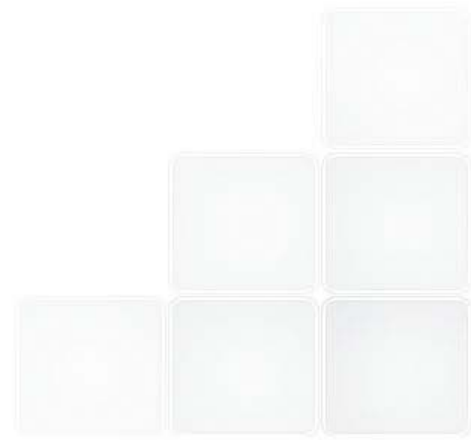
Evaluation of long-term behaviour of conditioned salt wastes

- **Characterization of the final waste forms**
- **Static leaching tests up to 150 days**
- **Assessment of the main parameters which influence fission products release (pH, temperature, contact time)**
- **Determination of the interactions between host matrix and individual fission elements**



Characterization of the final waste forms

Density measurements;
Thermogravimetric analysis;
SEM-EDS;
Optical Microscopy;
FTIR;
XRD



Static leaching tests up to 150 days

Reference Method

C 1285-02 - Standard Method for Determining Chemical Durability of Nuclear, Hazardous, and Mixed Waste Glasses and Multiphase Glass Ceramics: the Product Consistency Test (PCT)

WG3: New Waste Type

Static leaching tests up to 150 days

Normalized Release

$$NL_i = \frac{c_i}{f_i \cdot (SA/V)}$$

Normalized Release Rate

$$NR_i = \frac{c_i}{f_i \cdot (SA/V) \cdot t}$$

Percentage of release

$$(\sum a_i / a_0) \cdot 100$$

WG3: New Waste Type

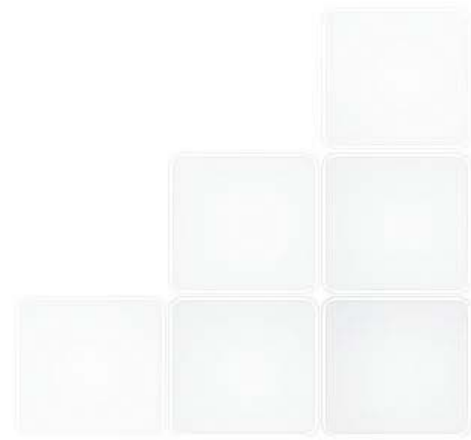


Assessment of the main parameters which influence fission products release (pH, temperature, contact time)

pH: acid; neutral; alkaline

Temperature: 23C; 90C

Contact time: 1; 3; 7 days



Determination of the interactions between host matrix and individual fission elements

Single fission elements incorporated in the host matrix
with characterization of the mineral phases formed:

Alkaline metal:

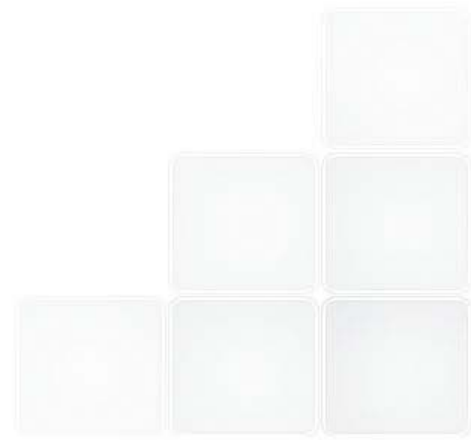
Cesium

Alkaline-earth metal:

Strontium

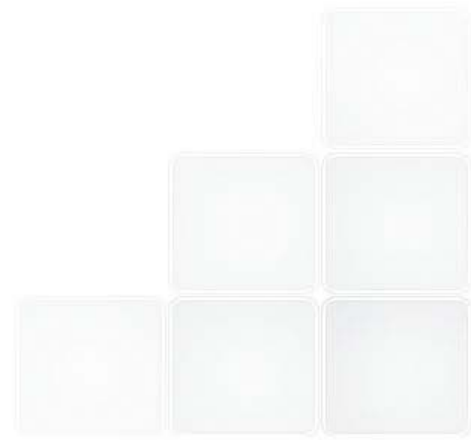
Rare earth metal:

Neodimium



CONCLUSIONS

The main scope of this proposal is a comparison among these promising matrices for conditioning of chloride salt wastes, with a particular attention to the incorporation mechanisms and to their durability in the long-term, in order to comply with geological disposal



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Thanks for your attention

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