



Novel treatments to improve radioactive waste disposal : the case of Andra

IGD-TP Exchange Forum n°6

4/11/2015

Andra, French Agency in charge of radioactive waste long-term management

- ◆ Independent from radioactive waste producers
- ◆ Under the supervision of several Ministries



Strengthen the overall consistency of waste management, from waste production until their disposal

Public endowment received in 2010 within the « Investissements d'Avenir » program to work ahead of disposal

- ◆ 75 M€ budget (almost over now)
- ◆ Goal: optimize radioactive waste disposal through innovative R&D projects on waste treatment
 - Optimize **waste disposal capacities** by volume reduction
 - Optimize **disposal safety** by processing waste to make them as inert as possible under disposal conditions
 - Optimize **waste take over and distribution between the different disposal facilities** already existing or planned

R&D collaborations between Andra and waste producers on waste processing

- ◆ R&D projects led by waste producers
- ◆ Andra brings the vision of disposal requirements and constraints



The PIVIC project



IL-LL waste only contaminated with alpha emitters

- ◆ Waste mainly arising from Areva facilities: glove boxes used for Mox production (Melox facility) and spent fuel reprocessing plants
- ◆ Mixed waste made of c. 30 % organic matter / c. 70 % metal & glass
- ◆ Production planned until 2042, c. 3,400 m³ on the whole



Original conditioning option (compaction) not accepted (2010)

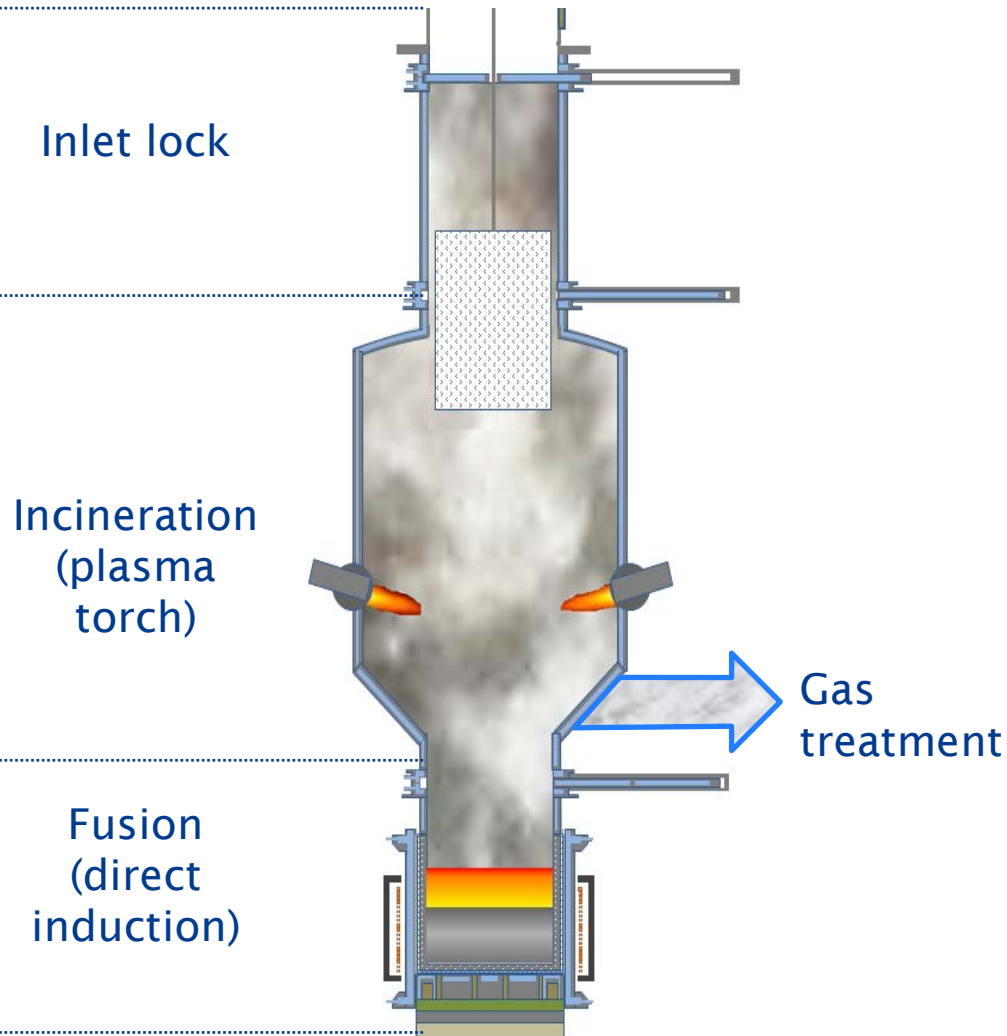
- ◆ Organic matter radiolysis and hydrolysis issues under disposal conditions
 - H₂ release → *overpressure, explosion issue*
 - Corrosive species release → *waste packages corrosion issue*
 - Complexing species release → *possible increase in some radionuclides mobility in disposal*



An alternative conditioning option is being studied, with the following requirements

- ◆ Full destruction of organic matter
- ◆ Confinement of radionuclides
- ◆ No pre-treatment
- ◆ Compactness
- ◆ Batch processing system





Process developed as a combination of pre existing technological elements

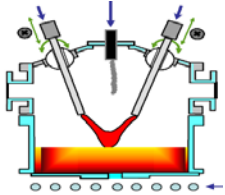
- ◆ Plasma incineration
- ◆ Vitrification of resulting ashes
- ◆ Fusion of metallic pieces
- ◆ In can process

Production of a biphasic waste

- ◆ Metallic fraction below
- ◆ Glass fraction above including ashes and actinides

Strong volume reduction

2011



Step 1. Exploratory research

- Fundamental concepts
- Final waste requirements



Step 2. Concepts feasibility

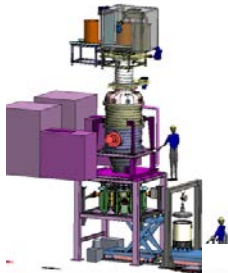
- Feasibility of waste conditioning material
- Feasibility of the process (fusion and incineration)
- Feasibility of gas treatment

2018



Step 3. Integration and development

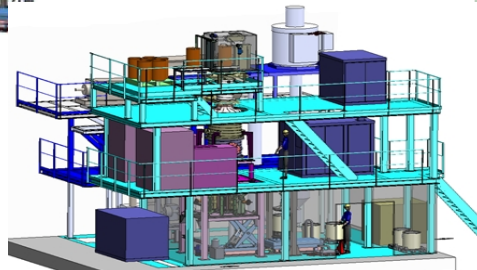
- Development of the full process scale 1
- Improvement of conditioning material
- Waste package description



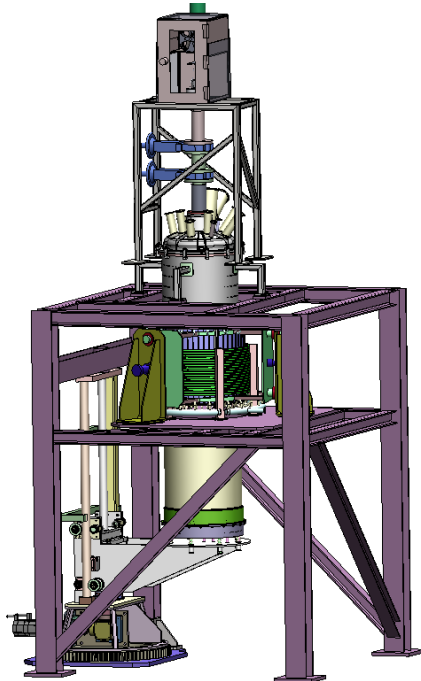
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Step 4 . Qualification

- Final process, to be nuclearized

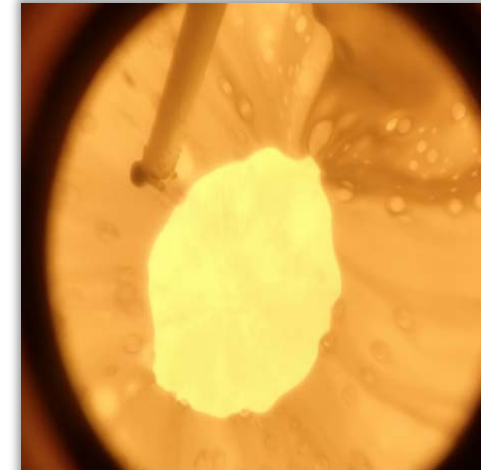


2024



Fusion pilot already built, tests underway

- ◆ Determination of operational parameters
- ◆ Study of the fusion of metallic pieces, including Al
- ◆ Study of ashes and actinides incorporation into the glass fraction
- ◆ Study of the phosphating process to manage metal chlorides
- ◆ Study of reactor fouling and actinide accumulation



Characterization of the first cans produced

- ◆ Very good separation observed between the metallic and the glass layers
- ◆ Actinides simulants almost exclusively located in the glass fraction



Graphite waste treatment



About 23,000 tons of LL-LL graphite waste

- ◆ 9 UNGG reactors (graphite moderated, fueled with natural uranium, CO₂ cooled)
- ◆ 3 main radionuclides of concern for interim storage and disposal : ³H, ¹⁴C, ³⁶Cl

Reference management solution in France: near surface disposal

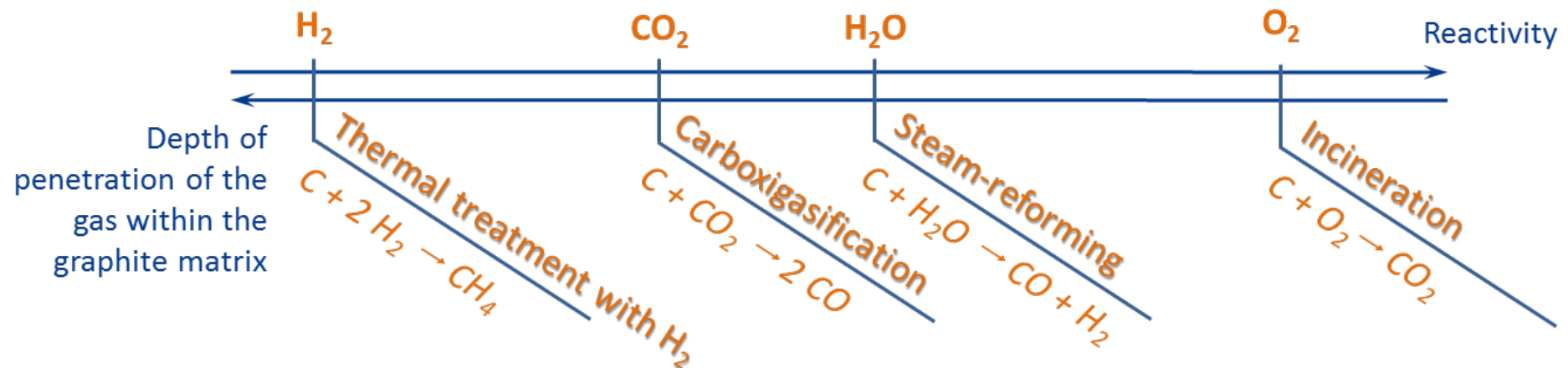
Graphite decontamination by thermal treatment studies

- ◆ Good performance for ³H and ³⁶Cl, low decontamination rates in ¹⁴C
- ◆ Discharges issues, complex management of secondary waste
- ◆ Strong lowering of graphite waste radiological inventory make treatment less interesting at this stage



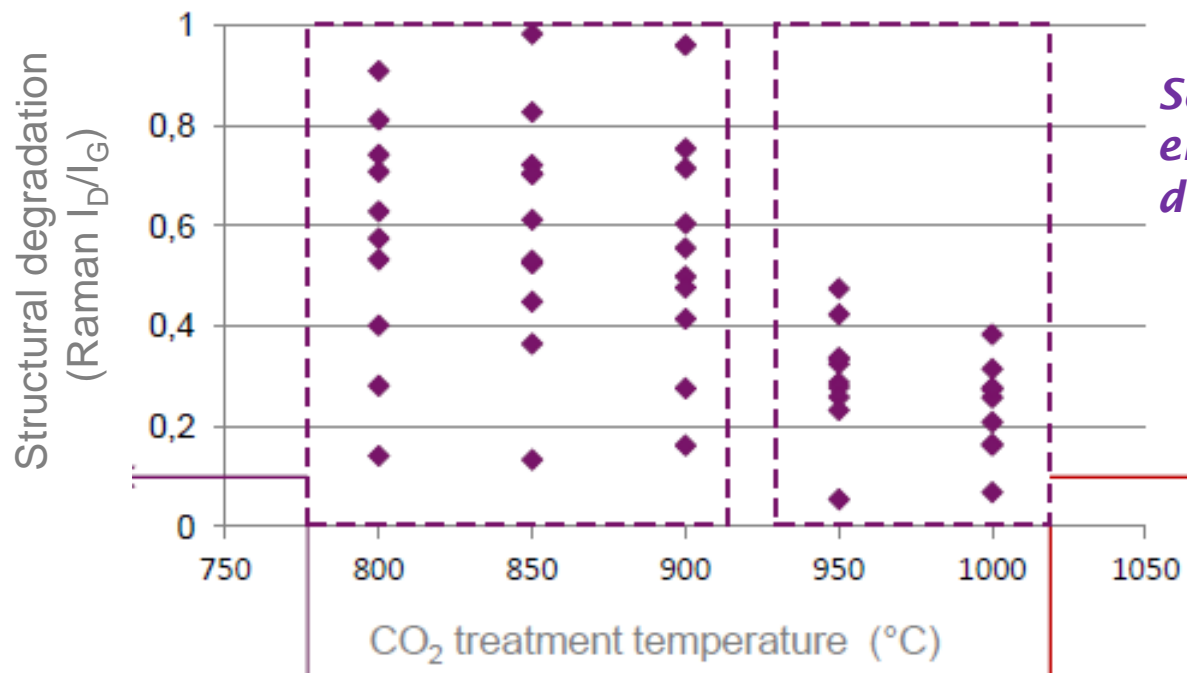
Basic principle: remove ¹⁴C in graphite bulk selectively from ¹²C

- ◆ Structural defects in graphite due to neutron flux are heterogeneous at the nanometer scale
- ◆ CO₂ will mainly react with degraded areas (more active sites)
- ◆ Assumption: ¹⁴C is mainly located in the most degraded areas of graphite matrix



CO₂ selectivity for degraded areas confirmed on i-graphite simulant (milled graphite) above 950 °C

PhD of Justin Pageot



Selective elimination of degraded particles

But moderated removal of ¹⁴C on i-graphite

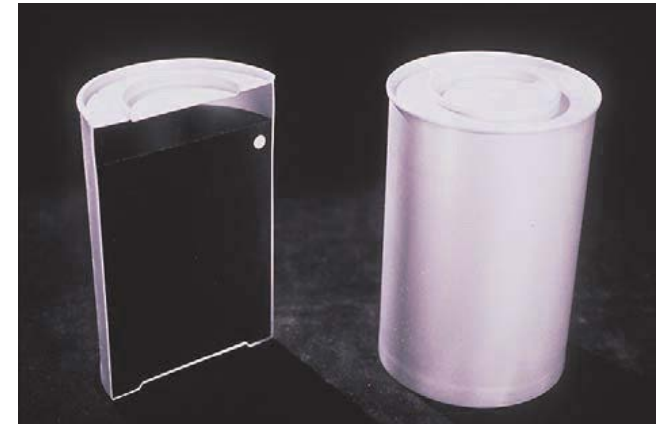
◆ 26 % of ¹⁴C removal for c.6 % weight loss at 950 °C



Research perspectives

About 20 000 m³ of bitumen waste in France

- ◆ c. 7,000 m³ LLW-LL + c. 13,000 m³ ILW-LL
- ◆ Sludge produced by effluents coprecipitation coming from spent fuel reprocessing
- ◆ Homogeneous waste, salts incl. BaSO₄, NaNO₃



Comprehensive R&D made on French bitumen characterization and behavior under disposal conditions (coll. Areva/CEA/EDF/Andra)

- ◆ Fire hazard control (self-heating, fire resistance)
- ◆ Better knowledge of bitumen variability composition
- ◆ Cracking hazard control (H₂ production, bitumen swelling)

Disposal is the reference solution for French bitumen waste management

Prospective R&D on bitumen treatment may be interesting to maintain alternative management solutions

- ◆◆ Bitumen long-term management options still open in some European countries
- ◆◆ Growing demand of civil society to dispose of non-reactive radioactive waste

Thermal treatment already studied at CEA but strong technological and scientific challenges remain

- ◆◆ Bitumen supply into the process
- ◆◆ Salts decomposition (BaSO_4) and redox reactions management
- ◆◆ Gas treatment
- ◆◆ Fissile material management

Launch prospective developments on bitumen treatment ?

- ◆◆ The 2016 Euratom call may be an opportunity